



# Toward the Ultimate Goal of Tritium Self-Sufficiency: Technical Issues and Requirements Imposed on Fusion Power Plants

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# Background Info

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- There is **no** practical **external source** of tritium (T).
- D-T fuelled fusion power plants **must breed** their own tritium.
- Plant with 1 GW fusion power **consumes huge amount of T** (55.6 kg per full power year).
- Shortage of T and surplus of T significantly impact plant operation and T storage.
- For licensing considerations, fusion should **not generate excess T** than needed.
- T bred in blanket should be accurately estimated as 1% uncertainty translates into 1-2 kg of T/FPY for 2-3 GW  $P_f$
- For advanced designs, **Net TBR** is close to 1.01.
- **Calculated TBR** must be greater than **Net TBR**.
- **Key questions:**
  - How high should **Calculated TBR** be? *Design and breeder dependent*
  - What elements determine breeding margin (**Calculated TBR**-1) ? *Four main elements*
  - Does this margin evolve with time? *Yes*
  - Could T breeding be adjusted online? *Yes, for liquid breeders through  $^6\text{Li}$  enrichment*
  - Could design over-breed or under-breed? *Less risky to over-breed*

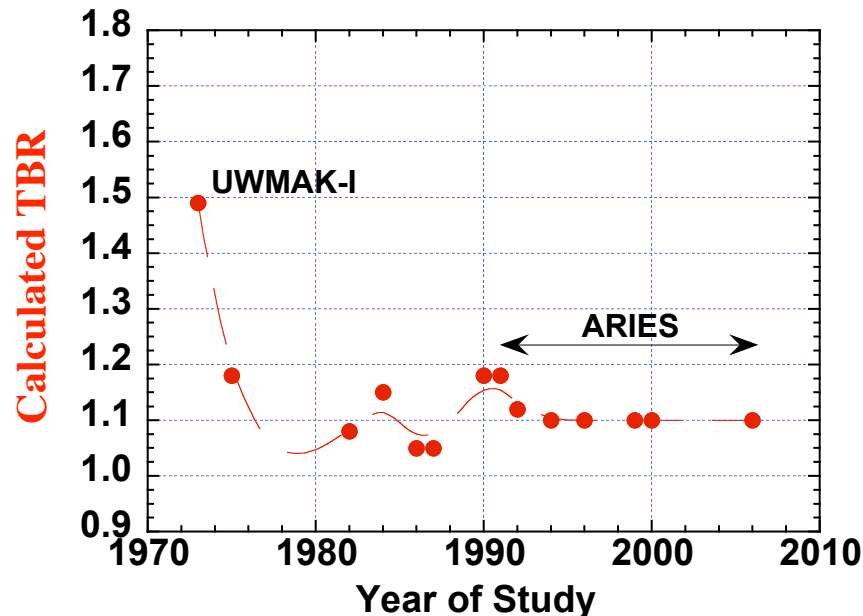


# Calculated TBR Evolves with Time and is Design and Breeder Dependent

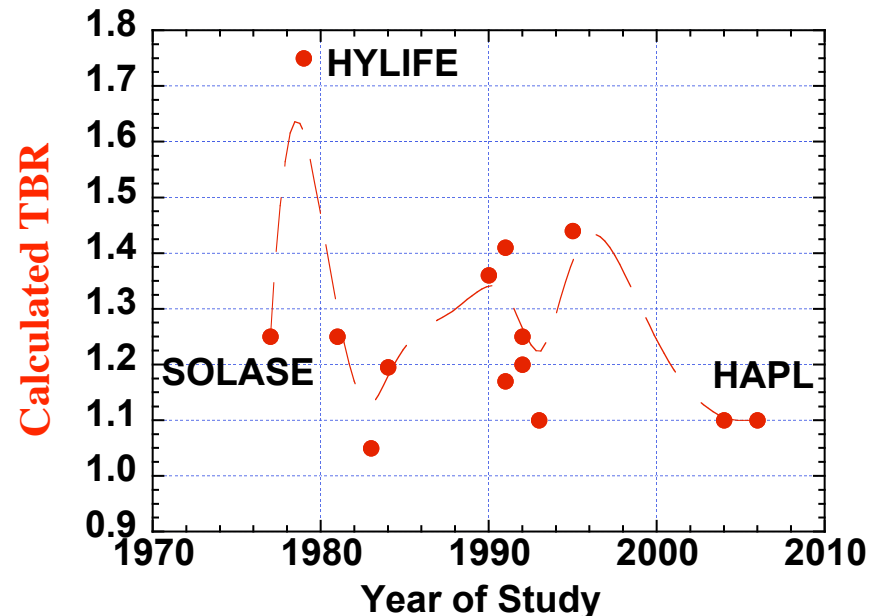
- There is **no general consensus** within fusion community on what the **Calculated TBR** should be.
- Advanced ARIES designs considered **Calculated TBR** of 1.1 for liquid breeders
- Other US projects (IFE **HAPL** @ NRL, Demo @ UCLA, IFE @ LLNL) along with some EU and J studies accord with ARIES 1.1 Calculated TBR.
- Some designs call for higher **Calculated TBR** with **Net TBR** of  $\sim 1.05$ .

## US Designs

### MFE



### IFE

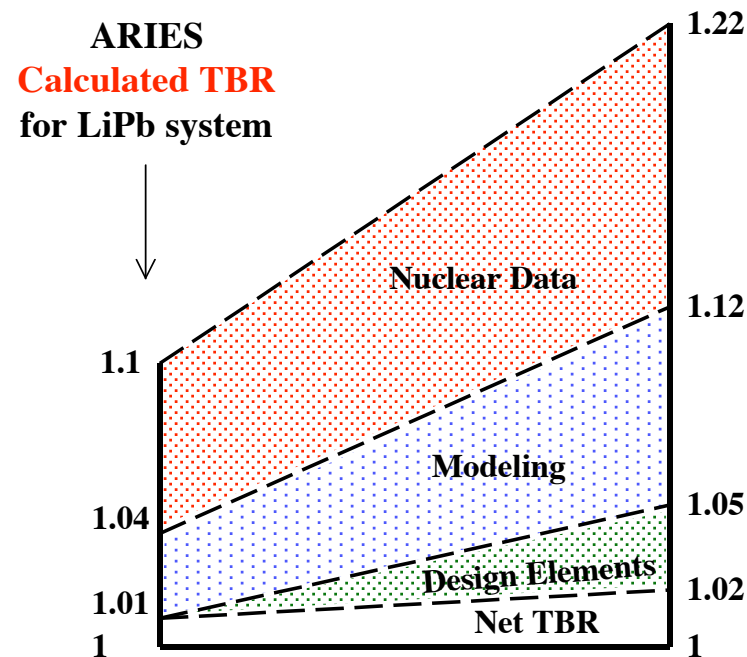


# Breeding Margin

(Calculated TBR – 1)

Can be divided into **4 distinct categories**:

- Margin for known deficiencies in nuclear data (6-10%) ← **breeder dependent**
  - Margin for known deficiencies in modeling (3-7%)
  - Margin for unknown uncertainties in design elements (0-3%)
  - Margin for T bred in excess of T consumed in plasma (1-2%)
- } **design dependent**





# Margin for Known Deficiency in Nuclear Data (6-10%)

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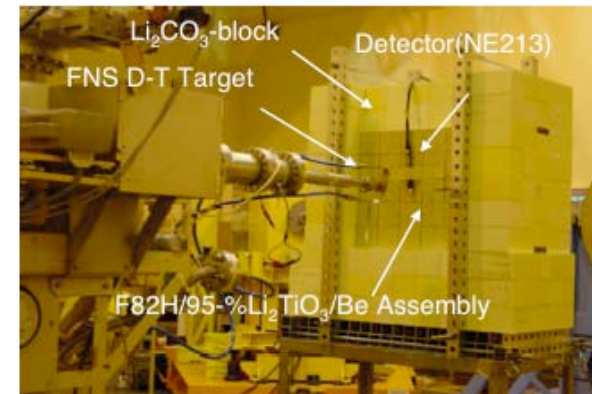
- T production is highly sensitive to neutron energy spectrum that is controlled by nuclear data evaluation for numerous isotopes (e.g., 20-30 isotopes in ARIES blankets) and cross-sections, not only (n,t).
- Several organizations in US, EU, and J developed nuclear data libraries for fusion applications.
- IAEA FENDL library is widely used worldwide as data were carefully selected from several national libraries.
- Despite high fidelity in IAEA evaluation, FENDL-2.1 version is far from perfect. Issuing new version takes years of extensive experimental program combined with data re-evaluation, then data validation.
- Impact of uncertainties in nuclear data evaluation on calculated TBR was assessed numerically @ UCLA for several breeders (~6% for LiPb).
- Few integral experiments (with 14 MeV neutron source) exist in J and EU to validate nuclear data.

# Margin for Known Deficiency in Nuclear Data (Cont.)

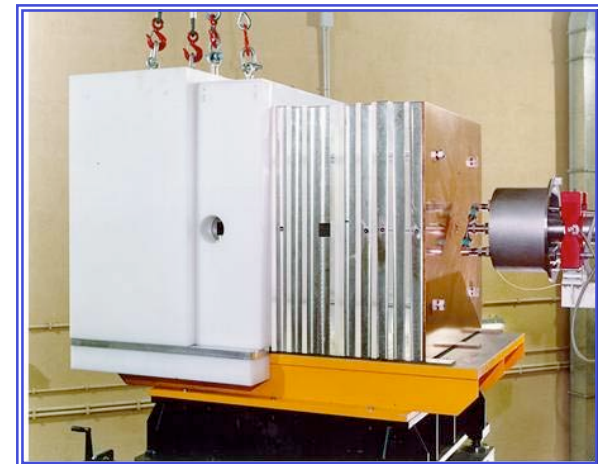
## Solid breeder experiments:

- Recent **FNS** results for  $\text{Li}_2\text{TiO}_3/\text{Be}/\text{FS}$  blanket indicated calculations overestimate T production rate by up to 10-20%.
- **FNG** experiment indicated T production is predicted within 5-10% uncertainty for solid breeding blankets with Be multiplier.

## **FNS Facility** (JAEA, Japan)



## **FNG Facility** (ENEA, Italy)





# Margin for Known Deficiency in Nuclear Data (Cont.)

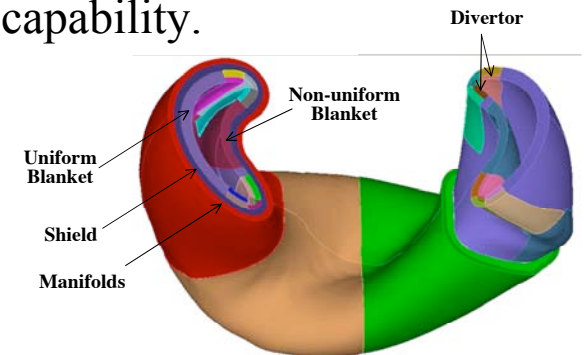
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- New experiments are underway in Japan and Europe for helium-cooled **LiPb** blanket (more relevant to ARIES).
- Several iterations between data evaluation and experimental validation will continue until good agreement is reached.
- ARIES will continue including adequate breeding margin (~6%) in **Calculated TBR** of LiPb system to account for nuclear data deficiency until J and EU conduct LiPb experiments, benchmark, and publish results.
- Only ITER TBM and CTF will accurately measure T production in prototypical fusion environment (neutron spectrum, surrounding components, etc.), contributing to validation of TBR calculations.

# Margin for Known Deficiency in Modeling (3-7%)

- **Calculating TBR** for any fusion system requires advanced neutronics tools.  
Newly developed CAD-MCNPX approach provides such capability.

- Ideally, 3-D model should include essential components that impact breeding significantly: FW, blanket, divertor, stabilizing shells, penetrations, and assembly gaps.



**Example: UW CAD/MCNPX approach applied to ARIES-CS**

- Practically, 3-D model cannot represent real geometry, particularly for a complex blanket designs as very detailed blanket is too costly to model.
- Homogenization overestimates breeding level and 3-D **Calculated TBR** should be adjusted accordingly.
- Margin of error in Calculated TBR due to modeling could range between 3 and 7%, depending on how crude 3-D model is.





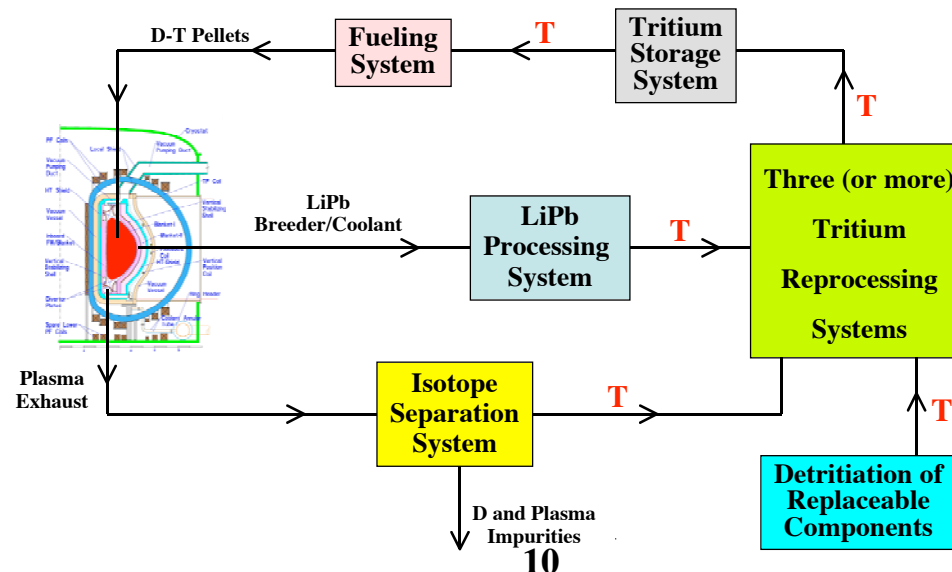
# Margin for Unknown Uncertainties in Design Elements (0-3%)

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- Normally, TBR is calculated for conceptual designs where major elements that degrade breeding (such as FW, blanket structure, and penetrations) are included in 3-D model.
- As design develops further approaching construction phase, several **future design changes** may negatively affect breeding, calling for larger breeding margin during conceptual phase.
- Such changes include:
  - Adding few mm W armor on FW to enhance plasma performance
  - More supporting structure for FW and blanket
  - Thicker SiC insulator for DCLL blanket concept
  - Larger stabilizing shells
  - Sizable penetrations
  - Wider assembly gaps.
- In ARIES, no provision was made to account for future design changes.
- Such changes will require higher enrichment and/or redesigning blanket to meet breeding requirement.

# Margin for T Bred in Excess of T consumed (1-2%)

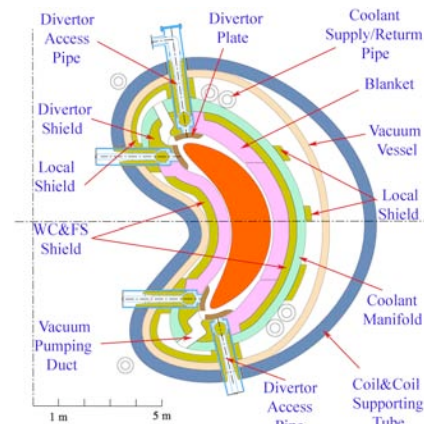
- Divided into three main categories:
  1. T required to **provide start-up inventory for new fusion power plant**:
    - a. T build-up in power core materials (especially in breeder, multiplier, structural materials) and T recovery system for blanket
    - b. T build-up in fuel reprocessing system (especially in cryo-panels, getters, molecular sieves)
    - c. T build-up in detritiation systems for coolants, building atmosphere, and vacuum pumping system
    - d. T to be stored in getters as reserve to continue plasma operation in case of temporary malfunctions of T reprocessing system
  2. T necessary to **compensate for decay** of total T inventory
  3. T **lost to environment** (atmosphere, cooling water, etc.).



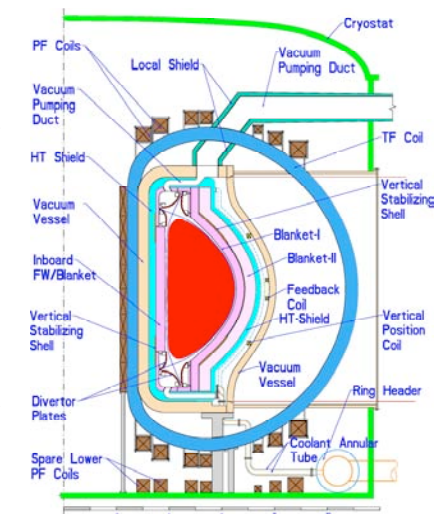
# Margin for T Bred in Excess of T consumed (Cont.)

	ARIES-CS	ARIES-AT
Net output power (MW <sub>e</sub> )	1000	1000
Fusion power (MW)	2436	1759
Burn-up fraction of T in plasma	12.4%	36.4%
T consumption: in kg/FPY	135	97.8
in kg/day	0.37	0.268
T throughput (kg/day)	3	0.74
T holdups in LiPb breeder (kg)	0.1	0.1
T holdups in structure (kg)	~1	~0.8
T inventory in reprocessing system (kg)	1.5	0.37
T build-up outside FPC (kg)	0.5	0.5
Stored T for malfunctions (kg)	1	0.25
T decay (kg/y)	0.33	0.16
T losses to environment (g/y)	< 4	< 4
Start-up inventory (kg)	~4	~2

ARIES-CS  
(Compact Stellarator)

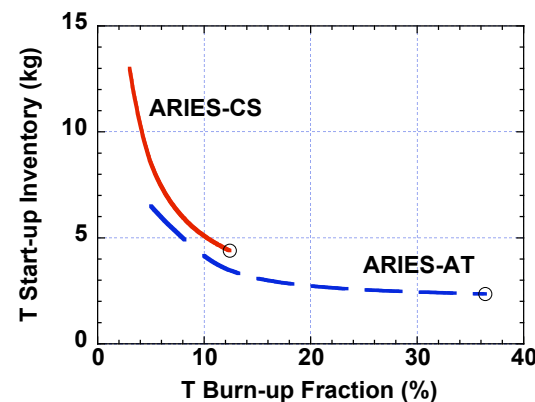


ARIES-AT  
(Advanced Tokamak)

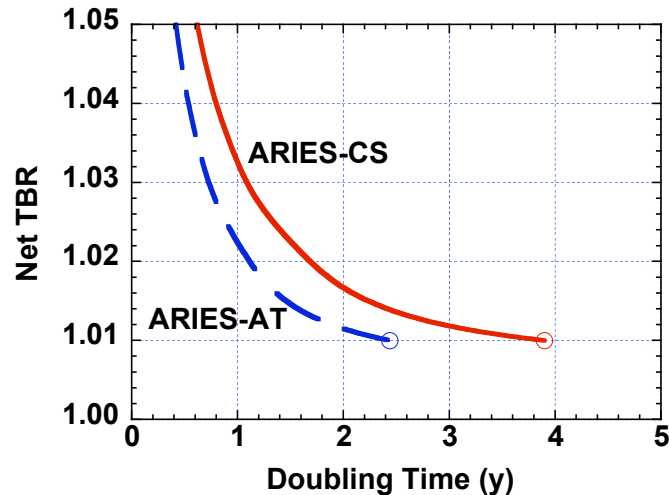


## Realistic assumptions:

- 3 fuel reprocessing systems
- 1 day T reserve to allow unperturbed plasma refueling
- Doubling time < 5 y



# Net TBR



**Doubling time** needed to supply new power plant with start-up T.  
**Mature fusion plants** call for **Net TBR**  $\leq 1.01$ .  
**Early generations** of fusion plants require **Net TBR**  $> 1.01$  with shorter doubling time.

**Advanced physics and technology help keep Net TBR around 1.01**

Essential requirements include:

T burn-up fraction in plasma exceeding 10%

High reliability and short repair time (< 1 day) for T processing system

Three or more T processing system

T and  $\alpha$  particles recycled at high rates

Low T inventory in all subsystems

Extremely low T losses to environment (< 4 g/y).

# Over-Breeding or Under-Breeding?

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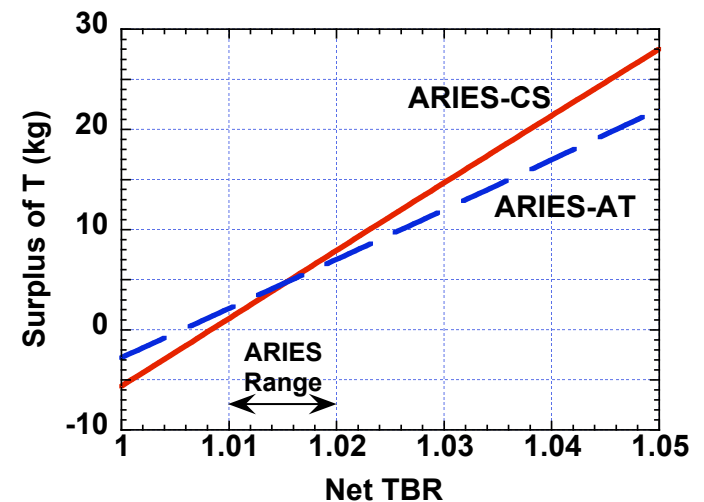
- Net TBR will not be verified till after Demo operation with fully integrated blanket and T extraction and processing systems.
- Existing blanket will be redesigned accordingly.
- All blankets should be flexible and accept few changes to deliver a **Net TBR** of 1.01.
- **Over-breeding blanket** (**Net TBR** > 1.01):
  - For liquid breeders, most practical solution is to adjust the  ${}^6\text{Li}$  enrichment online,
  - For ceramic breeders, adjust  ${}^6\text{Li}$  enrichment after first blanket change-out or replace few breeding modules by shield.
- **Under-breeding blanket** (**Net TBR** < 1.01):

Major design changes anticipated to raise TBR, unless reference blanket designed with  ${}^6\text{Li}$  enrichment < 90%:

  - Thickening blanket,
  - Replacing W stabilizing shells of ARIES-AT by Al or Cu shells,
  - Lowering the structural content within the blanket,
  - Adding a beryllium multiplier to the blanket,
  - Increasing plasma aspect ratio,
  - Operating tokamaks in a single-null mode.
- It is **less risky to design over-breeding blanket** (with **Net TBR** of 1.01 - 1.02) and develop feasible scheme to adjust breeding shortly after plant operation.
- Surplus of T could be excessive if Net TBR exceeds 1.01.

# Excessive Breeding Represents T Storage Problem

- **Without online adjustment of breeding,** surplus of T generated over blanket lifetime (~5 y) would be significant if Net TBR exceeds 1.01 (after subtracting start-up inventory for new plant (with 5 y doubling time) and account for T decay).
- For comparison, total T accumulated from all CANDU reactors reaches ~30 kg by 2025.
- T breeding should be controlled with accuracy better than 1% to ensure T self-sufficiency without storage problem for surplus of T





# Proposed Scheme for Online Adjustment of **LiPb** Breeding

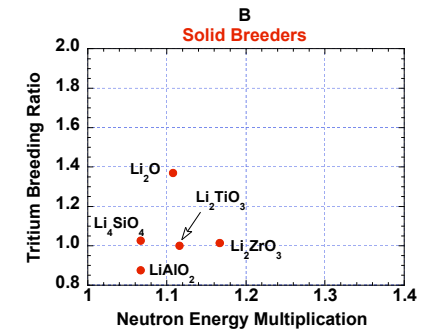
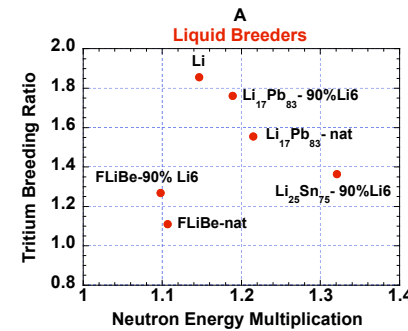
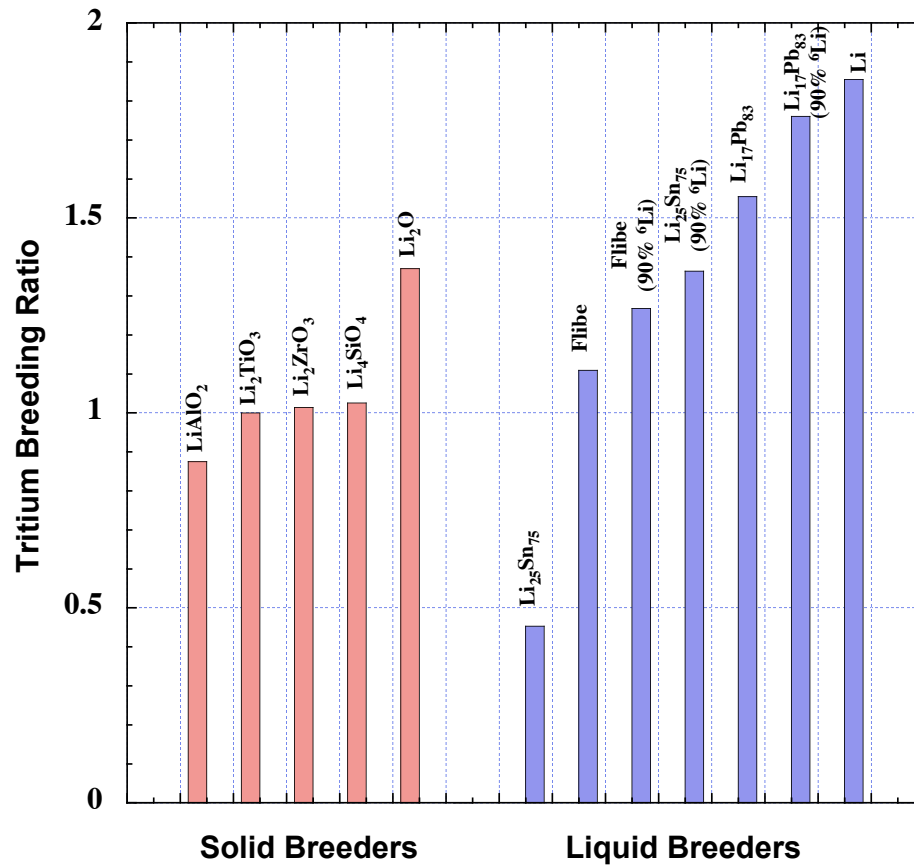
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- Two practical methods are feasible through combining two LiPb eutectics with different enrichments:
  - a) Replace X tons of enriched LiPb by X tons of LiPb with 100%  $^7\text{Li}$**   
(straightforward but requires additional storage volume for eutectic).
  - b) Remove Y\* tons of enriched Li from LiPb eutectic and replace it with Y tons of  $^7\text{Li}$**   
(does not require large storage, but needs practical method to remove enriched Li from eutectic and feed Li back with 100%  $^7\text{Li}$ ).

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\*  $Y \ll X$

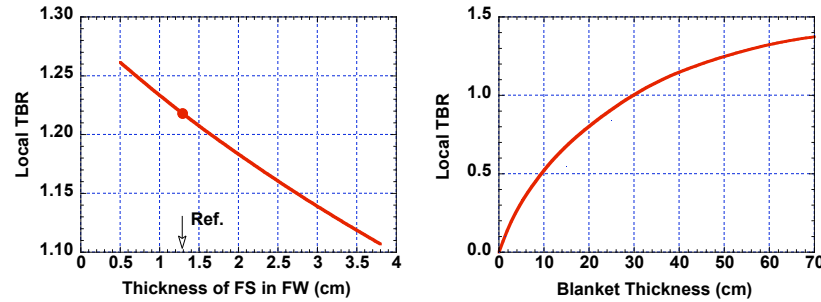
# Impact of Design Elements on Breeding Capacity



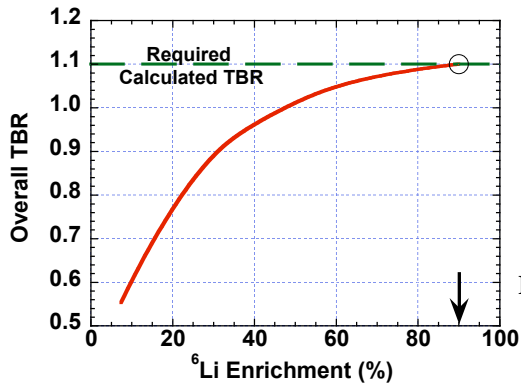
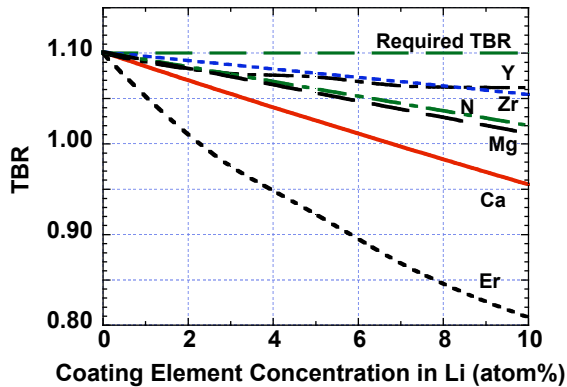


# Impact of Design Elements on Breeding Capacity (Cont.)

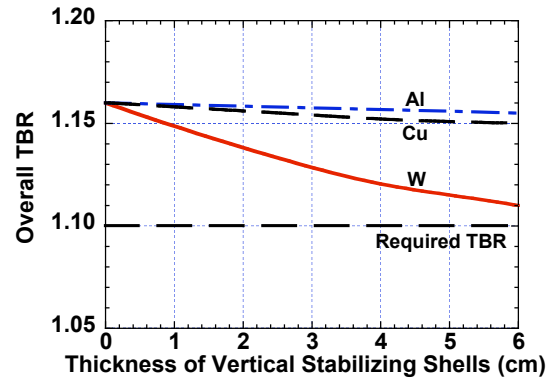
DCLL Blanket



ARIES-RS  
V Coating for  
Li/V Blanket

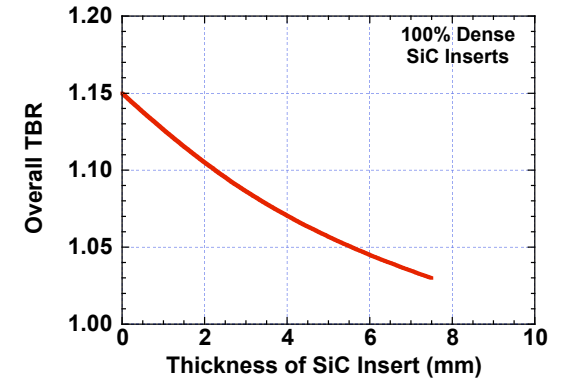


ARIES-AT  
Enrichment

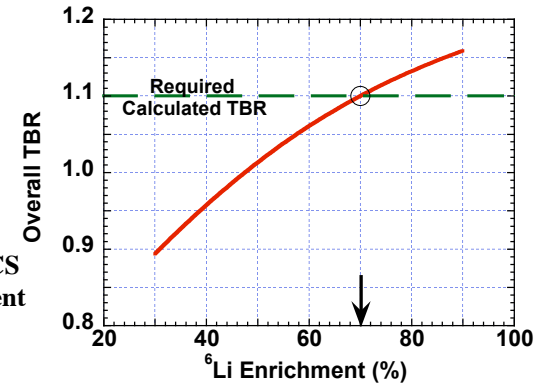


ARIES-AT Stabilizing Shells  
Located between LiPb/SiC  
Blanket Segments

ARIES-AT DCLL Blanket



ARIES-CS  
Enrichment



# Conclusions

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- **Breeding margin** (calculated TBR-1) is **breeder and design-dependent**. It **evolves with time** and accounts for known deficiencies in calculated TBR due to data and 3-D modeling, unknown uncertainties in design elements, possible malfunctions during plant operation, and start-up T supply for new power plant.
- 3-D **Calculated TBR** must be greater than **Net TBR**. 1% uncertainty translates into 1-2 kg T/FPY (for 2-3 GW fusion power), causing shortage or surplus of T.
- Net TBR must be very close to unity in order to ensure sufficient T supply without generating excessive T surplus.
- **Dedicated R&D program** will reduce breeding margin.
- **LiPb** blanket parameters should be determined for  **${}^6\text{Li}$  enrichment < 90%**.
- **Online adjustment of breeding is *must* requirement** for Demo and power plants.
- Such online adjustment is **feasible for liquid breeder blankets** through adjusting  ${}^6\text{Li}$  enrichment of breeder, but **difficult to envision for solid breeder blankets**.