



ARIES-ACT-DCLL NWL Distribution and Revised Radial Build

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Neutron Wall Loading Distribution



ARIES-ACT-DCLL Parameters (aggressive physics)

Fusion Power	2767.5 MW
Major Radius	6 m
Minor Radius	1.5 m
Elongation	2.2

Plasma surface area = 565 m²

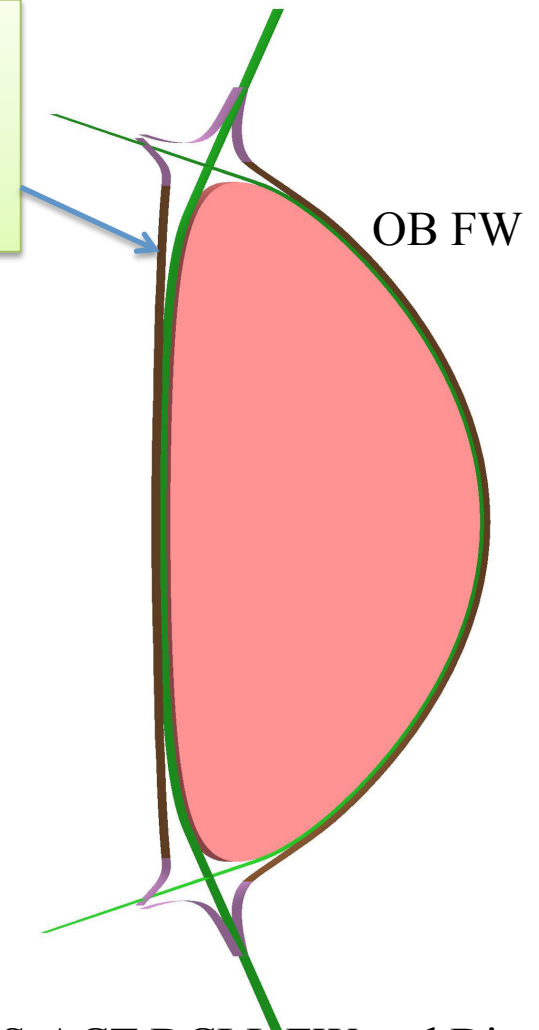
Surface area @ 10 cm from plasma = 588 - 599 m²

Average NWL @ 10 cm from plasma = 3.7 – 3.77 MW/m²

OB FW area = 350 m²

IB FW area = 180 m²

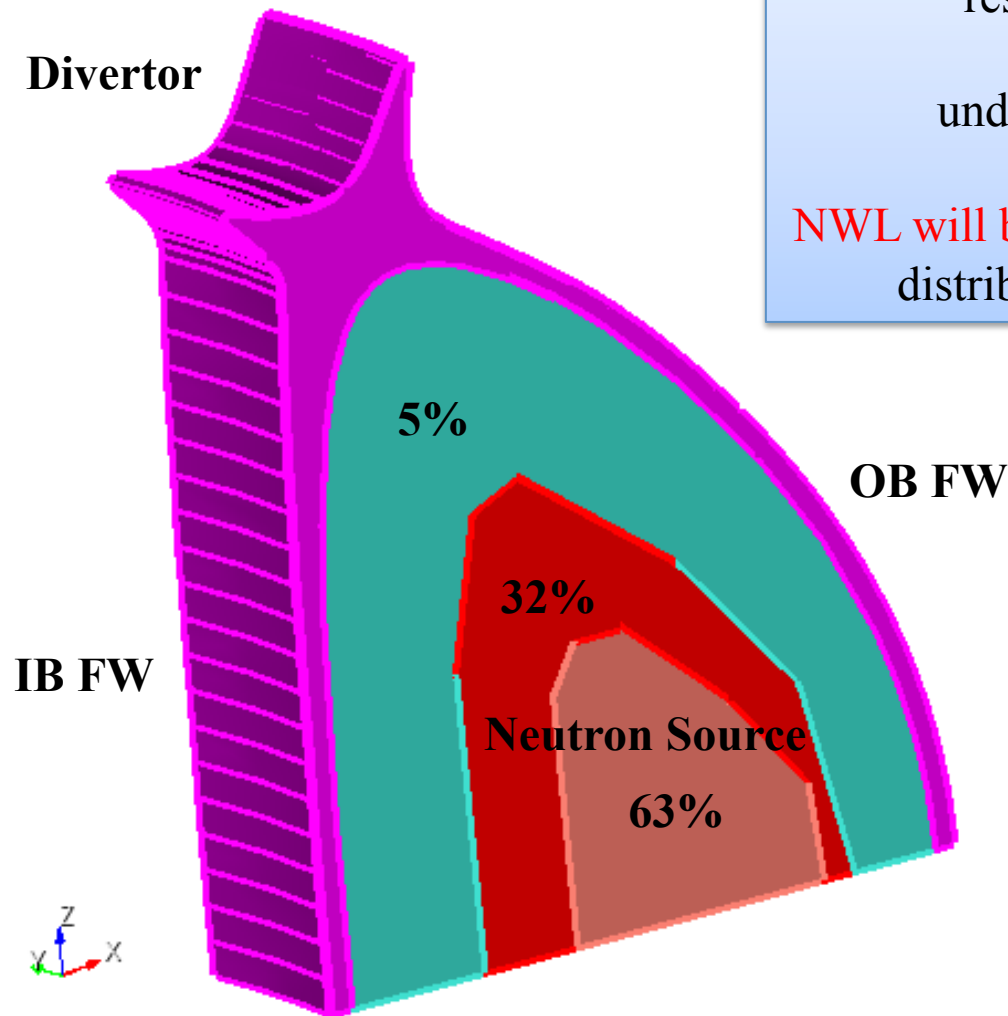
Curved IB FW offers
10 cm more space at
top/bottom for
IB manifolds



ARIES-ACT-DCLL FW and Div
(Courtesy of X. Wang (UCSD))



Neutron Wall Loading 3-D Model



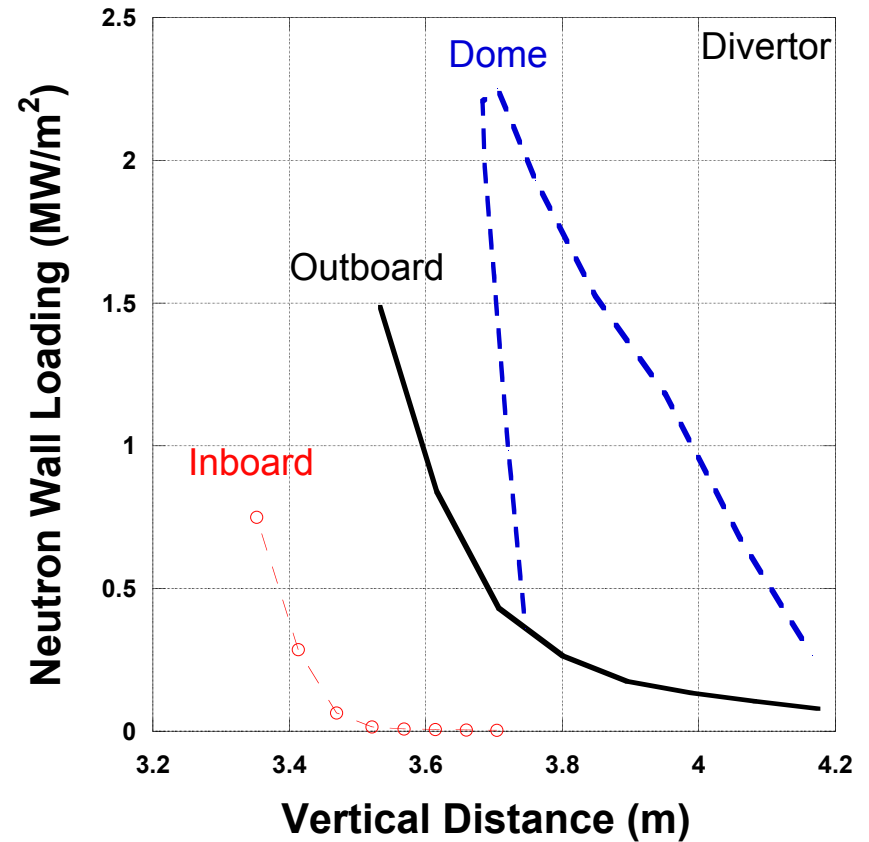
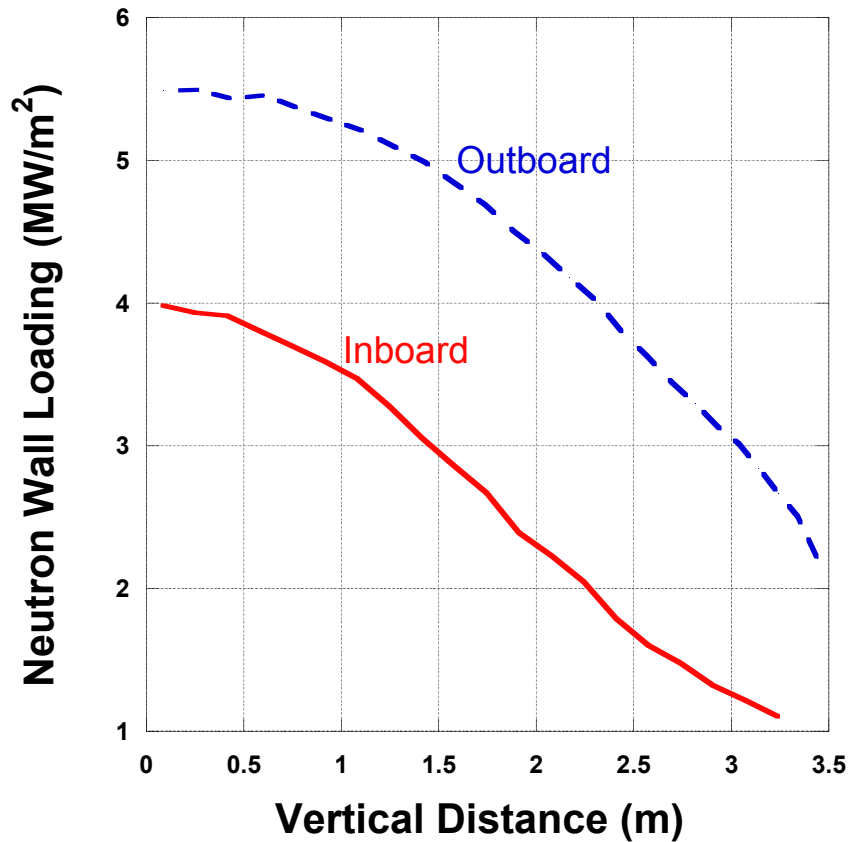
Approximate neutron source intensity

results in exact OB and divertor
NWL distribution, but
underestimates peak IB Γ by 10%.

NWL will be updated using actual neutron source
distribution on R-Z grid within plasma



NWL Distribution



Peak Γ @ IB $4 \times 1.1 = 4.4 \text{ MW/m}^2$
 Peak Γ @ OB 5.5 MW/m^2
 Peak Γ @ Div 2.2 MW/m^2

Peak to av. NWL ≈ 1.5

Too high for DCLL blanket, per S. Malang.

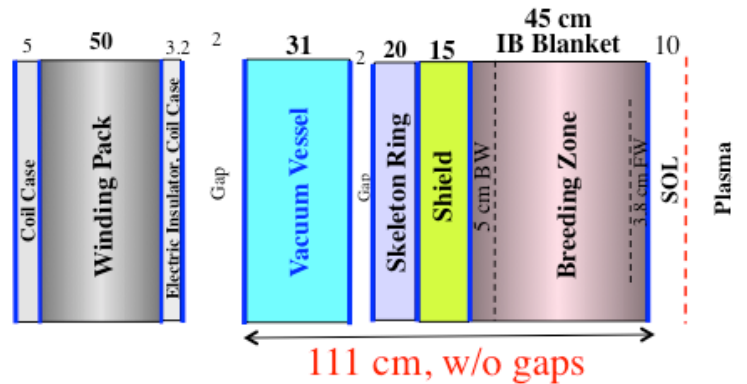
Revised Radial Builds:

- **Inboard**
- **Outboard**
- **Divertor.**

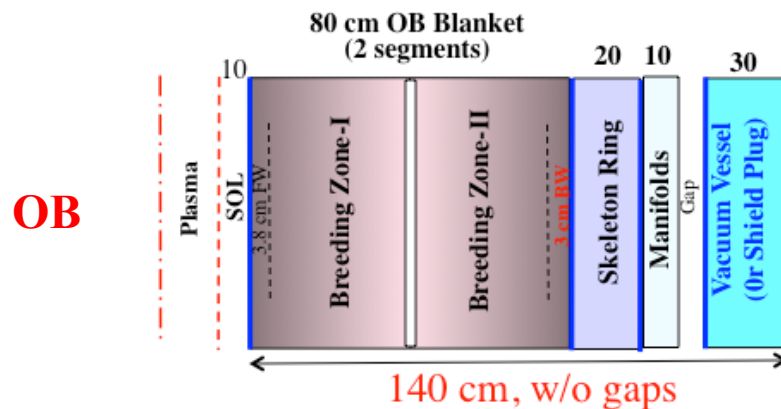


Initial Radial Builds

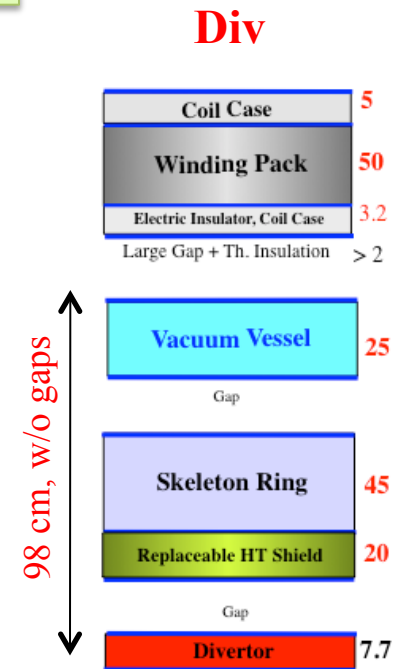
2009/2010 Radial Builds



IB (most compact radial build)



OB



Thicker blankets (0.65 m on IB and 1 m on OB) mandate redefining thicknesses and compositions of non-breeding components to satisfy design requirements and radiation limits.



Major Changes/Additions to Radial Builds

- Thicker **IB Back Wall** to protect Skeleton Ring for plant life:
 - ⇒ no need for replaceable HT shield.
- Reweldable back of **Skeleton Ring** (at least 20 cm thick).
- **Reweldable VV** everywhere including IB. (None of previous ARIES design had reweldable IB VV at midplane).
- Exclude materials with high **decay heat** to control temperature during LOCA:
 - ⇒ no WC filler in IB VV
 - ⇒ no W (or WC) for Shielding Block behind IB assembly gaps.
- 2 cm thick **He-cooled Thermal Shield** between VV and magnets.

Changes lead to larger radial builds



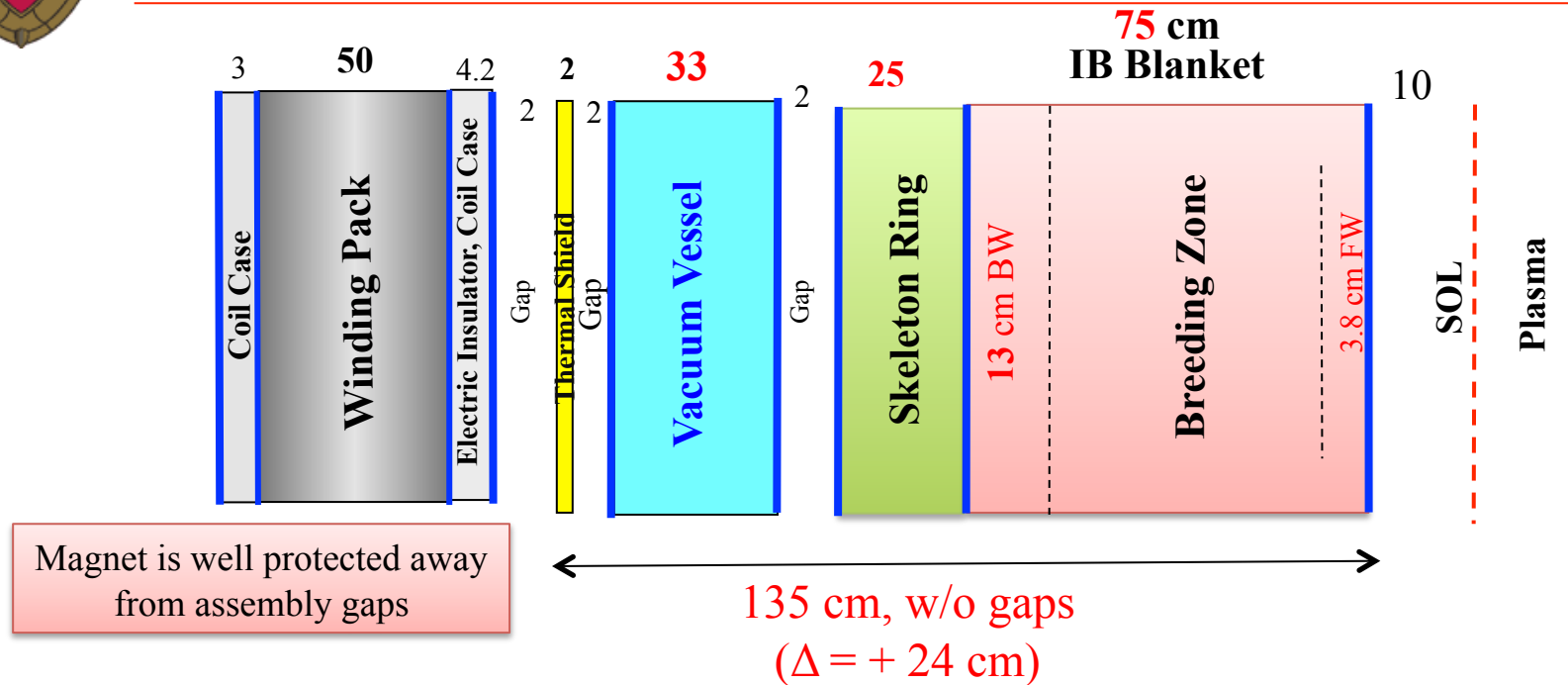
ARIES-ACT-DCLL Radiation Limits

Calculated Overall TBR	1.1	
Net TBR (for T self-sufficiency)	~1.01	
Damage to Structure (for structural integrity)	200 ???	dpa - advanced FS W structure
Helium Production @ Manifolds & VV (for reweldability of FS structure)	1	He appm
LT S/C Magnets (@ 4 K):		
Peak fast n fluence to Nb ₃ Sn ($E_n > 0.1$ MeV)	10^{19}	n/cm ²
Peak nuclear heating	2	mW/cm ³
Peak dpa to Cu stabilizer	6×10^{-3}	dpa
Peak dose to GFF polyimide insulator	$< 10^{11}$	rads
Plant Lifetime	40	FPY
Availability	85%	
Operational Dose to Workers and Public	< 2.5	mrem/h



ARIES-ACT-DCLL **IB** Radial Build

(Peak IB $\Gamma = 4.4 \text{ MW/m}^2$)



Vacuum Vessel:

2 cm thick face sheets.

Space in between filled with:

15% steel structure, 25% B-steel, 60% water.

Permanent component (16 dpa peak).

Reweldable (1 He appm peak).

Impact of n streaming through gaps TBD.

Skeleton Ring:

20% FS, 20% He, 60% B-FS.

Permanent component (185 dpa peak).

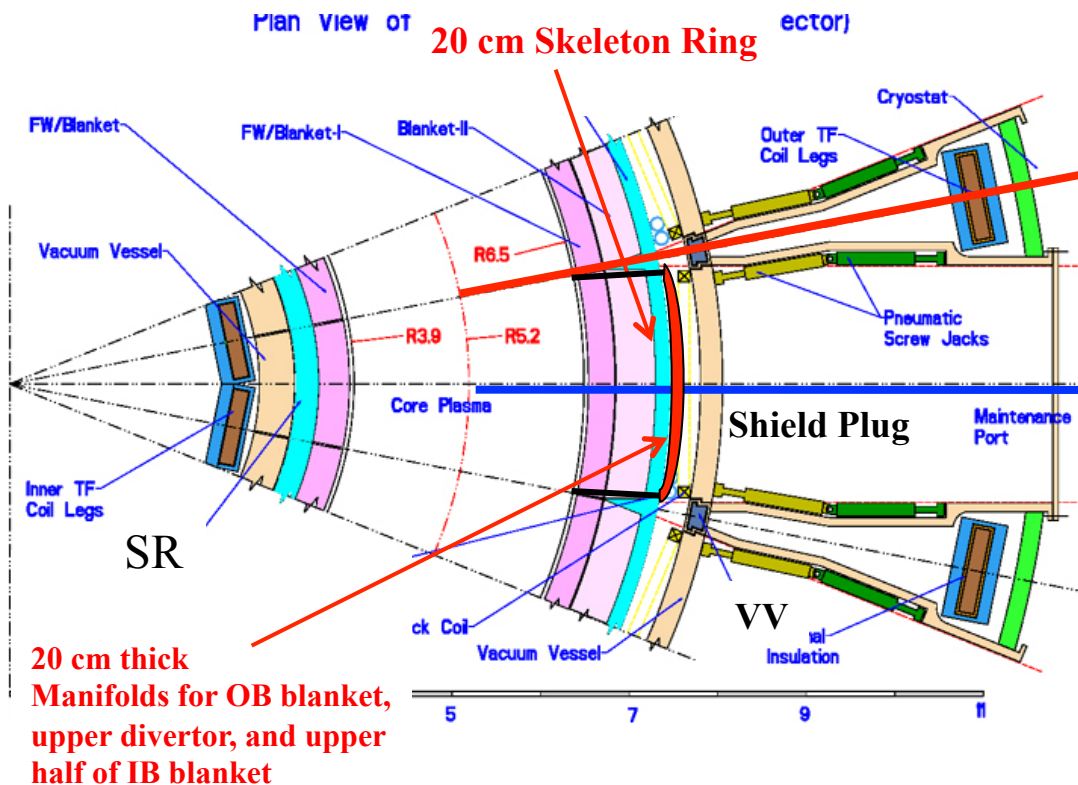
Reweldable at back.

Impact of n streaming through gaps TBD.



Two OB Cross Sections

ARIES-ACT



Cross Section I underneath magnet:

Shield and VV only.

No blanket.

TBD with 3-D analysis.

Cross Section II between magnets:

Blanket, SR, mnflds, and Shield Plug.

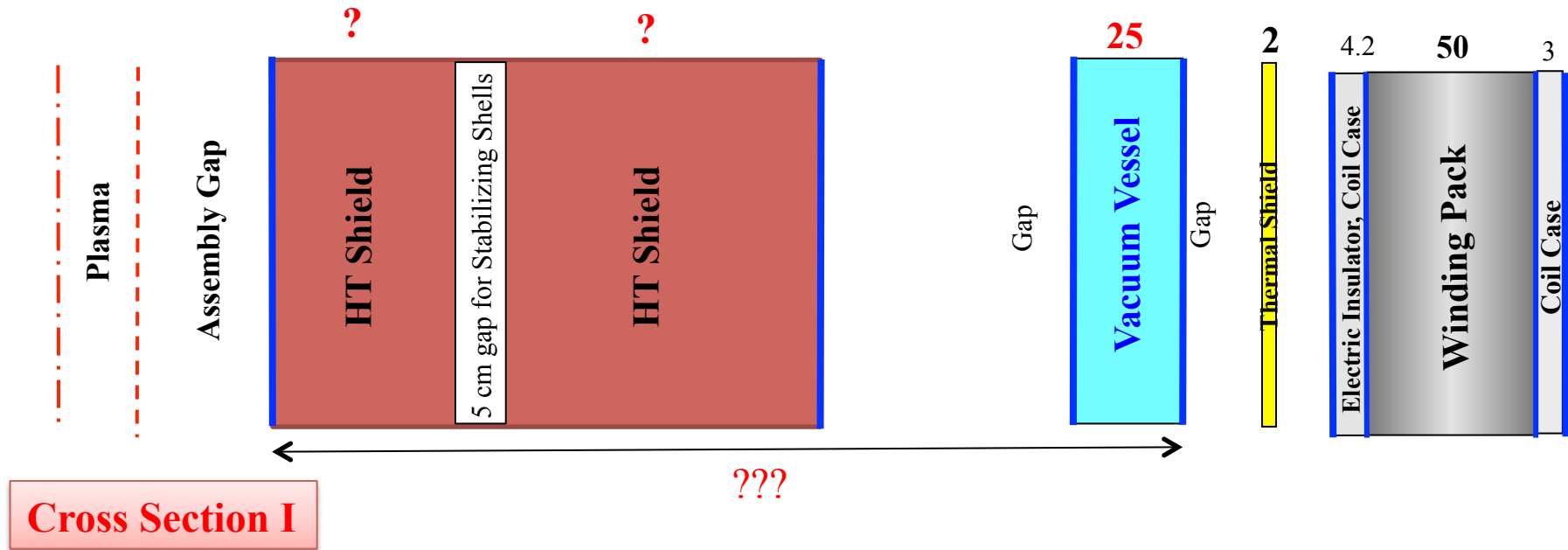
These components along with port walls protect sides of magnets.

This should be confirmed with 3-D analysis.



ARIES-ACT-DCLL **OB** Radial Build

Underneath Magnet (Peak IB $\Gamma = 5.5 \text{ MW/m}^2$)



HT Shield:

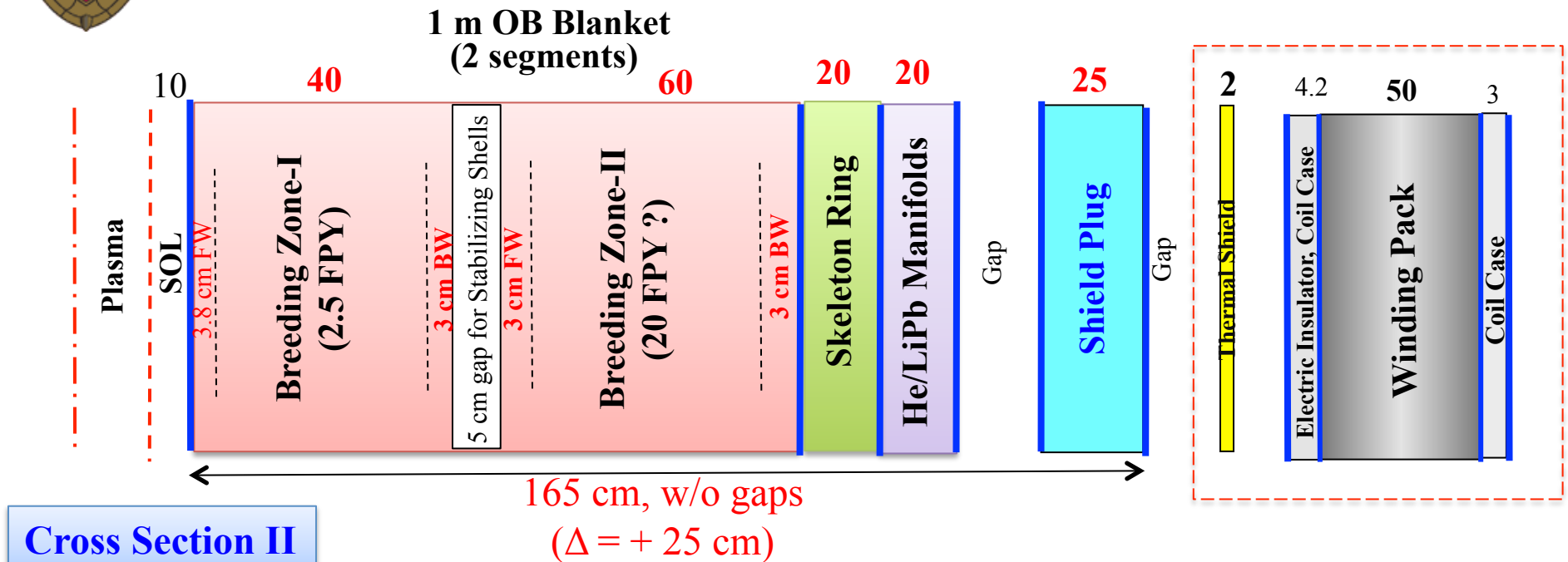
80% FS, 20% He, 60% B-FS
Replaceable and permanent components.
Thicknesses TBD with 3-D analysis to account for n streaming through assembly gaps.

Vacuum Vessel:

2 cm thick face sheets.
Space in between filled with:
15% steel structure, 25% B-steel, 60% water.
Permanent component?
Reweldable?
Impact of n streaming through gaps TBD.



ARIES-ACT-DCLL **OB** Radial Build Between Magnets (Peak IB $\Gamma = 5.5 \text{ MW/m}^2$)



Skeleton Ring/Manifolds:

SR: 80% FS, 20% He.

Permanent components (40 dpa peak).

Reweldable manifolds (0.2 He appm peak).

Impact of n streaming through gaps TBD.

Shield Plug:

2 cm thick face sheets (like VV).

Space in between filled with:

15% steel structure, 25% B-steel, 60% water

Permanent component (4 dpa peak).

Reweldable (0.08 He appm peak).

Impact of n streaming through gaps TBD.



ARIES-ACT-DCLL **Div** Radial Build

(Peak div $\Gamma = 2.2 \text{ MW/m}^2$)

Vacuum Vessel:

2 cm thick face sheets.

Space in between filled with:

15% steel structure, 25% B-steel, 60% water.

Permanent component (1.6 dpa peak)

Reweldable (0.34 He appm peak).

Impact of n streaming through gaps and pumping ducts TBD.

Skeleton Ring/Manifolds:

SR: 80% FS, 20% He, 60% B-FS.

Permanent SR (180 dpa peak).

Reweldable manifolds (1 He appm peak).

Impact of n streaming through gaps and pumping ducts TBD.

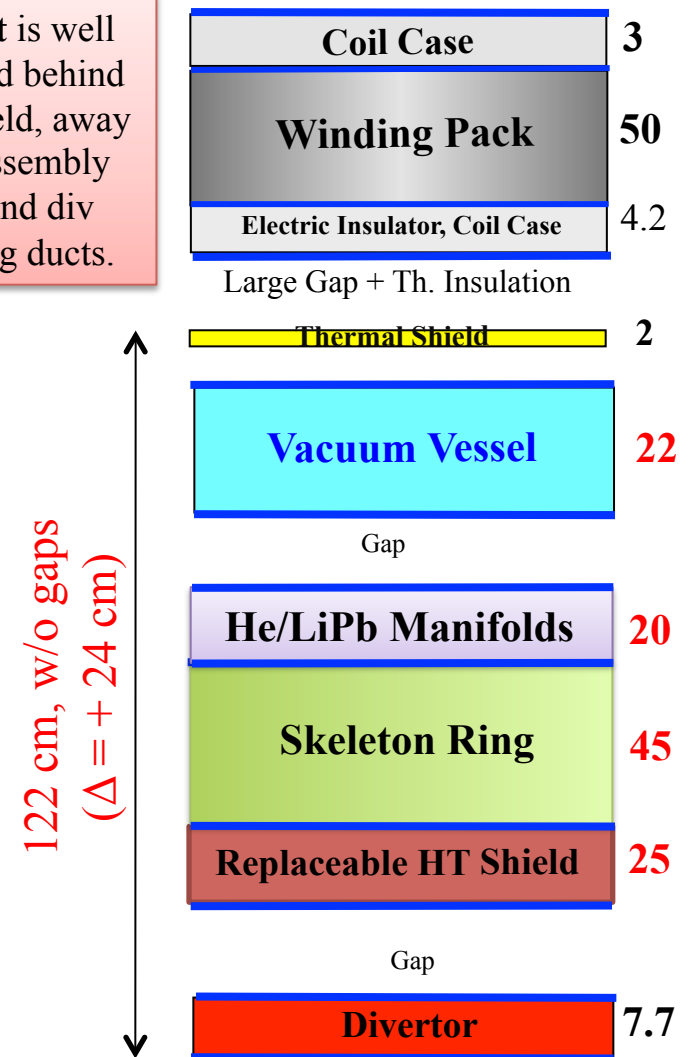
HT Shield:

80% FS, 20% He, 60% B-FS

Replaceable every 5 FPY.

Impact of n streaming on lifetime TBD.

Magnet is well protected behind bulk shield, away from assembly gaps and div pumping ducts.





IB Blanket Composition (75 cm thick)

	Thickness (cm)	Composition (volume %)
FW	3.8	8% ODS FS 27% MF82H FS, 65% He
Breeding Zone	58.2	77% LiPb (70% enriched Li) 7% MF82H FS, 3.7% SiC, 12.3% He/void (LiPb @ 580 °C; 9 g/cm ³ density; Li _{15.7} Pb _{84.3})
Back Wall	13	80% MF82H FS, 20% He
Alternate FW design with W	3.8	8.2% W, 8.3% ODS-FS, 22.4% MF82H FS, 61.1% He



OB Blanket Composition (1 m thick)

	Thickness (cm)	Composition (volume %)
Blanket-I: FW	3.8	8% ODS FS 27% MF82H FS, 65% He
Breeding Zone-I	33.2	77.2% LiPb (70% enriched Li) 7% MF82H FS, 4.2% SiC, 11.6% He/void (LiPb @ 580 °C; 9 g/cm ³ density; Li _{15.7} Pb _{84.3})
Back Wall	3	80% MF82H FS, 20% He
Alternate FW design with W	3.8	8.2% W, 8.3% ODS-FS, 22.4% MF82H FS, 61.1% He
Blanket-II: Front Wall	3	same as Back Wall
Breeding Zone-I	54	78.4% LiPb (70% enriched Li) 8.3% MF82H FS, 3.9% SiC, 9.4% He/void (LiPb @ 580 °C; 9 g/cm ³ density; Li _{15.7} Pb _{84.3})
Back Wall	3	80% MF82H FS, 20% He



Magnet Composition

	Thickness (cm)	Composition (volume %)
He-cooled Thermal Shield	2	20% steel, 80% He (@ 80-100 k)
Coil Case	3	95% JK2LB steel, 5% LHe
Winding Pack (from C. Kessel)	50	70% JK2LB steel, 13% Cu, 2% Nb ₃ Sn, 10% LHe, 2.5% GFF Polyimide

Magnet dimension and composition disagree with ASC ?!



VV Radiation Damage @ 40 FPY

(behind bulk blanket and shield, away from assembly gaps and penetrations)

@ Top/Bottom:

0.34 He appm

1.6 dpa

(not behind pumping ducts)

He/dpa = 0.2

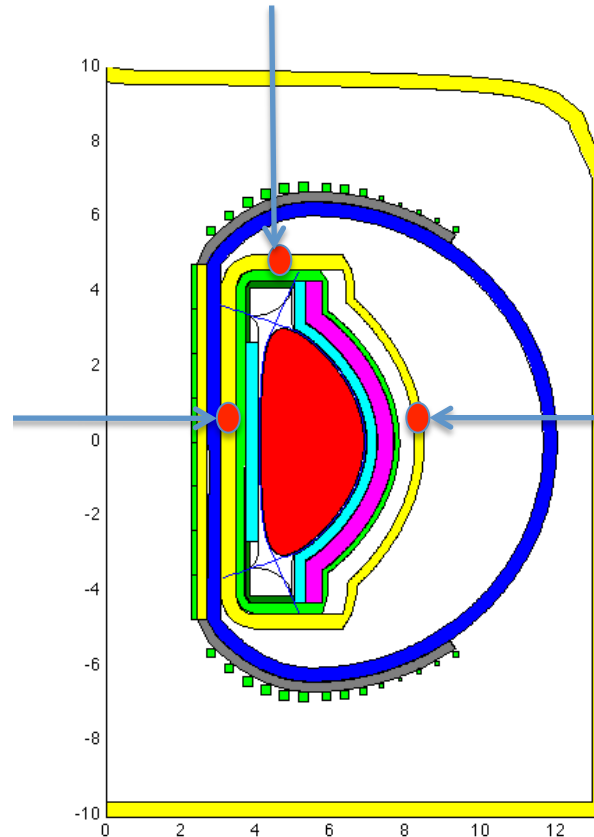
@ IB midplane:

1 He appm

16 dpa

(not behind assembly gaps)

He/dpa = 0.06



@ OB midplane:

? He appm @ VV

? dpa @ VV

He/dpa = ? for VV

@ Shield Plug:

0.08 He appm

4 dpa

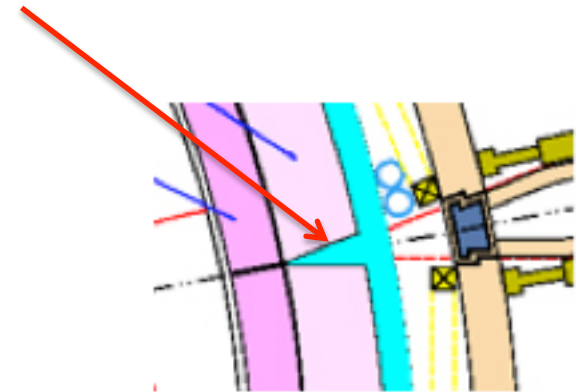
He/dpa = 0.02

This vertical cross section is misleading!
Only shield for Xn through VV and magnet.
NO VV and magnet behind OB blanket.



Streaming Concerns

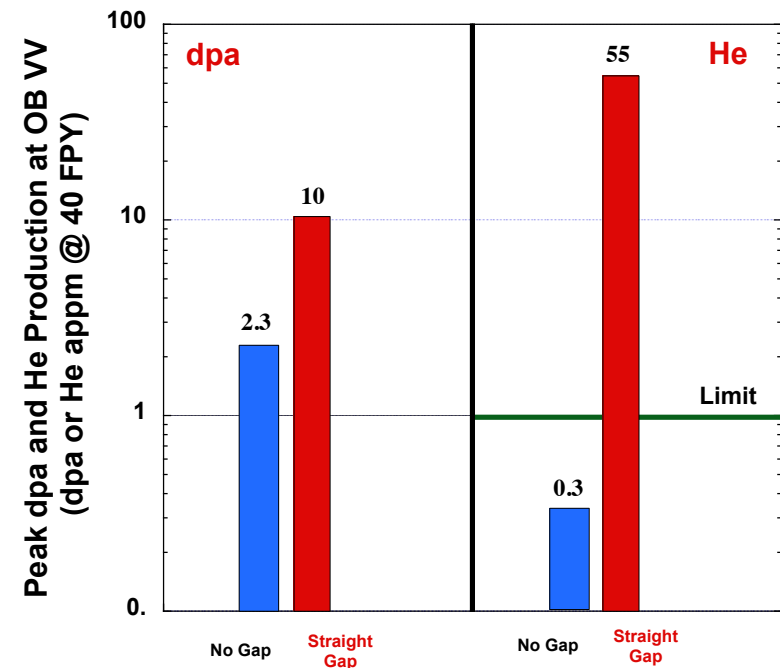
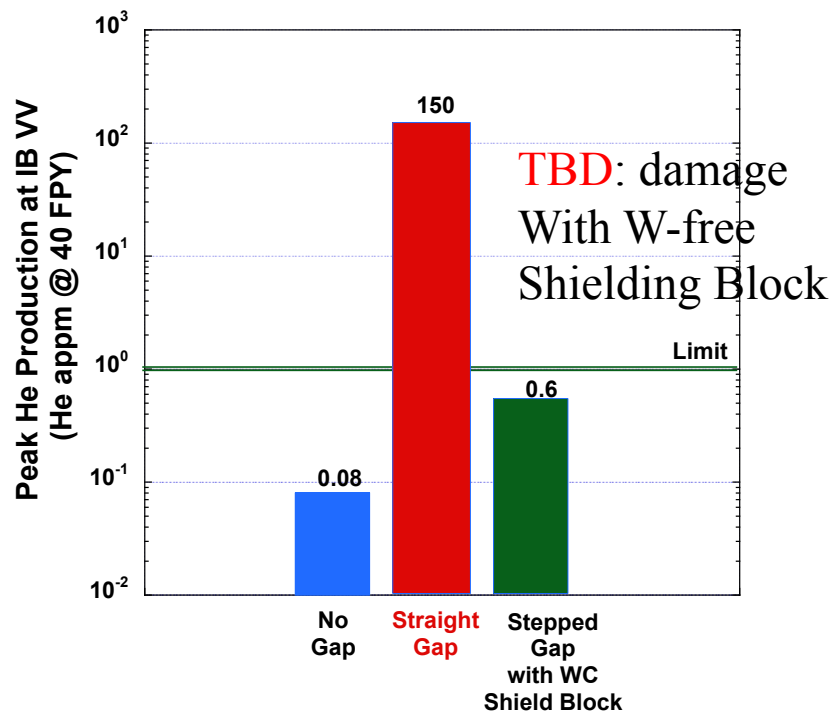
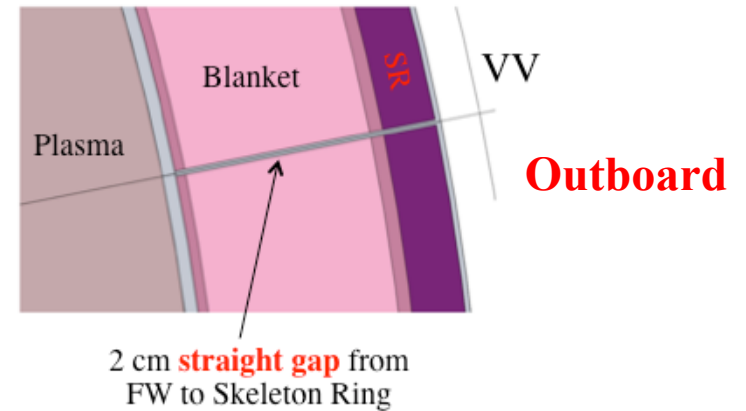
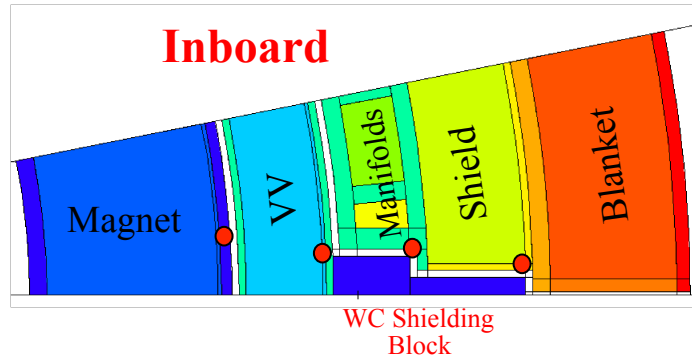
- With 3-D analysis, **effect of neutron streaming** on damage level should be assessed for:
 - Shield
 - Skeleton ring
 - Vacuum vessel
 - Stabilizing shell.
- **ARIES-ACT-DCLL has several streaming problems due to:**
 - **Assembly gaps:**
 - Horizontal/radial gap at IB midplane
 - Radial/poloidal assembly gaps between IB modules
 - Radial/poloidal assembly gaps between OB modules (blue wedged shape shield in ARIES-AT is **inefficient** and will be redesigned)
 - Radial/poloidal assembly gaps between divertor plates.
 - **Large penetrations:**
 - Plasma control
 - Fueling
 - Diagnostics
 - Divertor pumping ducts.





Examples of Neutron Streaming Effect

2 cm Wide Assembly Gaps increase He by > 1000 and dpa by > 4



Ongoing study



Final Remarks

- 3-D shielding/streaming analysis will be performed to:
 - Develop OB cross-section underneath magnet
 - Map radiation damage poloidally and toroidally, taking into consideration effect of neutron streaming through assembly gaps and large penetrations.
- FS-based designs should **avoid using** materials with relatively high decay heat (such as **W and Mn-based steels**) in order to control thermal response during transients (such as LOCA).
- Designing **W-free Shielding Block** behind assembly gaps is challenging!
- Seeking novel ideas to **conduct/radiate decay heat** to surroundings particularly for IB components.
- **Q:** type of **steel for ARIES VV and Thermal Shield?**