



Three Dimensional Analysis of Radiation Streaming Through ARIES-CS He-access Pipes

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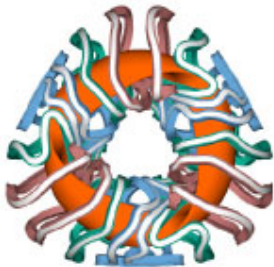
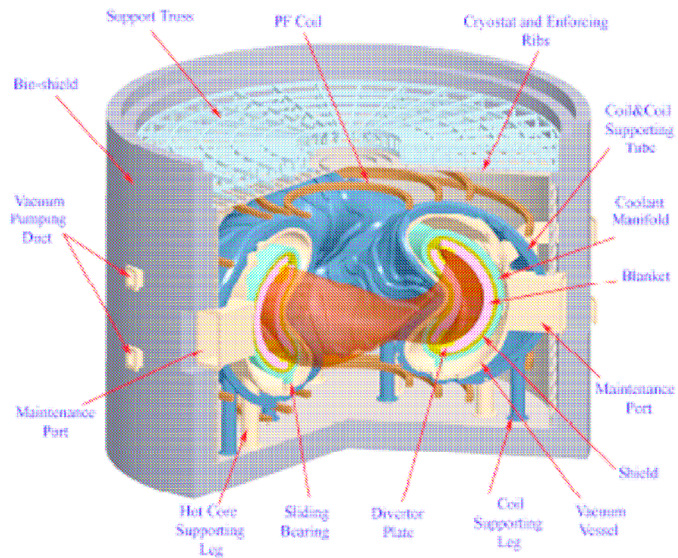


Outline

- Radial build of ARIES-CS
- 2-D analysis (simple straight pipe)
- 3-D analysis (pipe with shielding plug and inserts)
- Conclusions



ARIES- Compact Stellarator



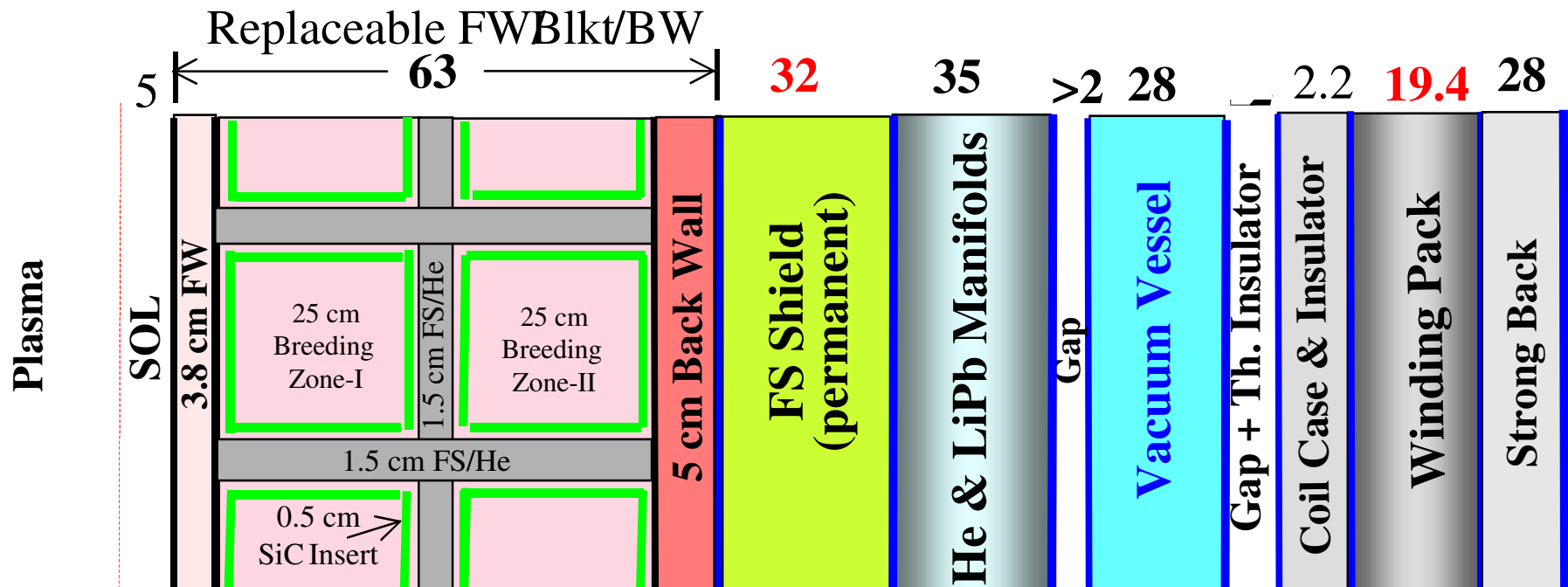
Plasma and poloidal coils

- ARIES-CS: a compact stellarator fusion power plant
- Parameters
 - 1000 MW net electrical power
 - 5.3 MW/m² Peak NWL
 - 40 FPY plant lifetime
- The device has a 120° symmetry in the toroidal direction



Radial build

Radial builds and radiation levels were defined in 1-D analysis without penetrations.





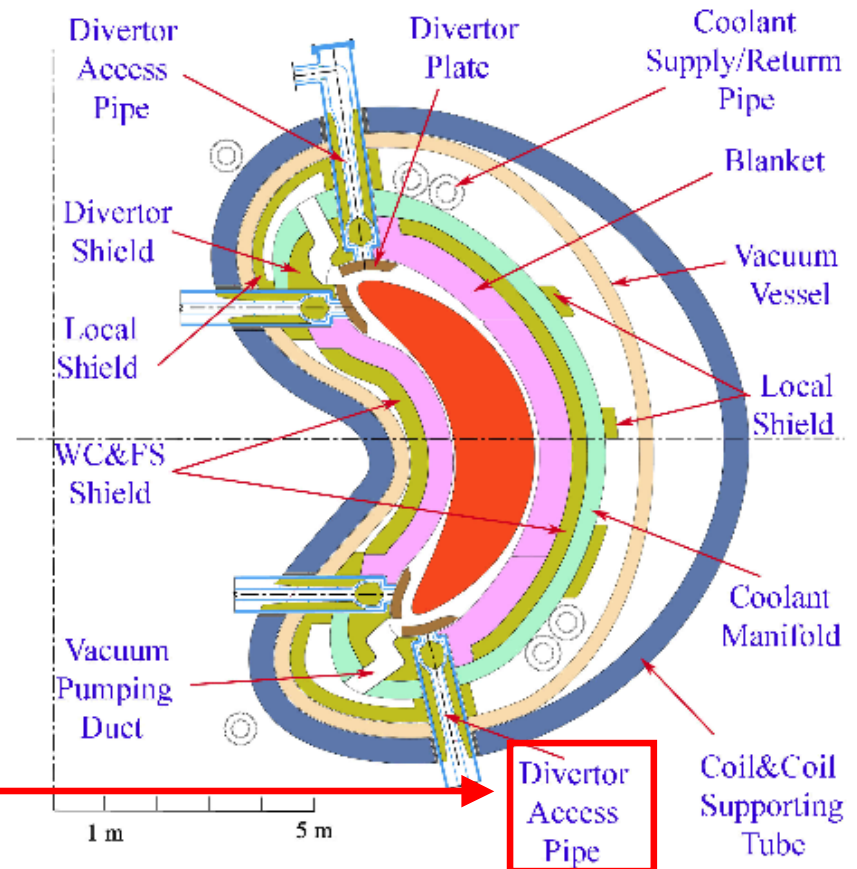
Fluxes and Damage Levels in Absence of Penetrations

	Nominal Values (without pipes)	Design Limits	Allowable Peaking Factor
dpa at shield (dpa @ 40 FPY)	135	200	1.5
He production at manifolds (He appm @ 40 FPY)	1	1	1
He production at VV (He appm @ 40 FPY)	0.2	1	5
Fast neutron fluence @ magnet (n/cm ² @ 40 FPY)	0.2×10^{19}	1×10^{19}	5
Total neutron flux outside magnet (n/cm ² .s)	10^7	5×10^7	5



Penetrations

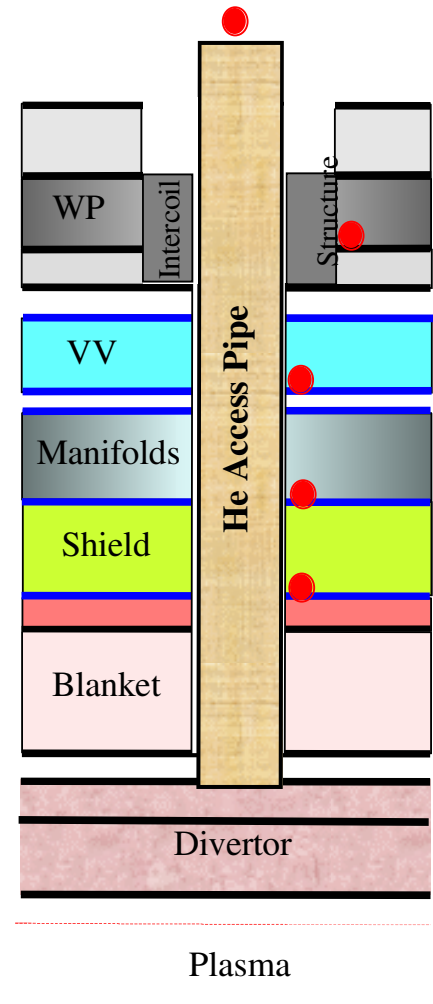
- Necessary:
 - vacuum pumping
 - coolant supply lines
 - plasma control
 - maintenance ports.
- Compromise shielding performance:
 - Enhance damage at surrounding components.
 - Increase dose levels behind magnet.
- Large divertor He-access pipe represents the most serious





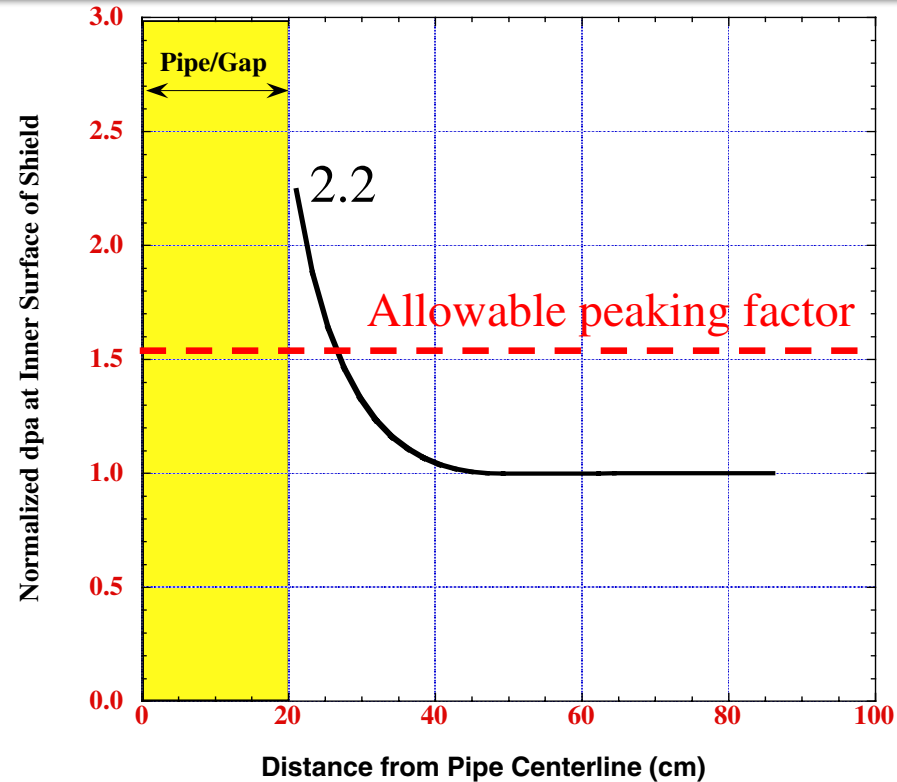
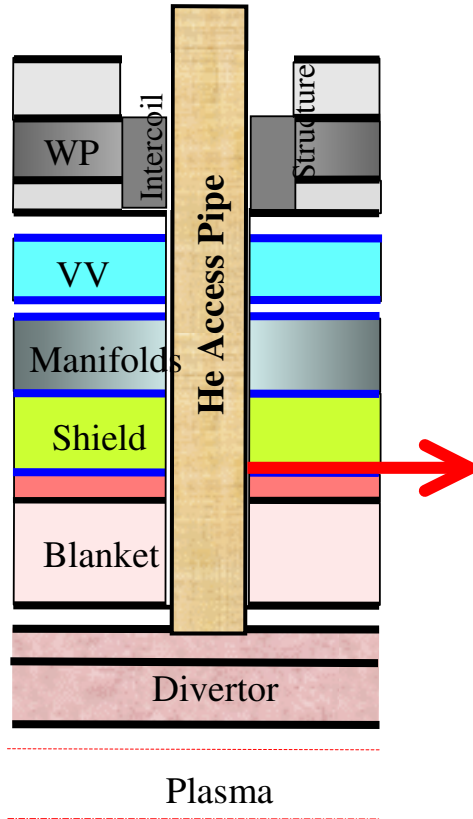
2-D Analysis (Simple Pipe with 30 cm ID)

- DANTSYS - a multi-dimensional, discrete ordinates transport code, developed by Los Alamos National Laboratory
- Data - FENDL2.1, 175n - 42g coupled cross section library





2-D Results (Shield)



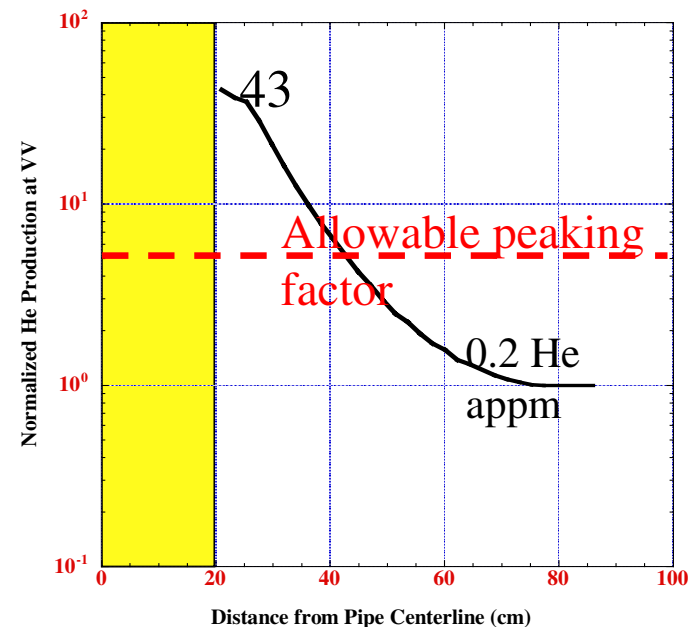
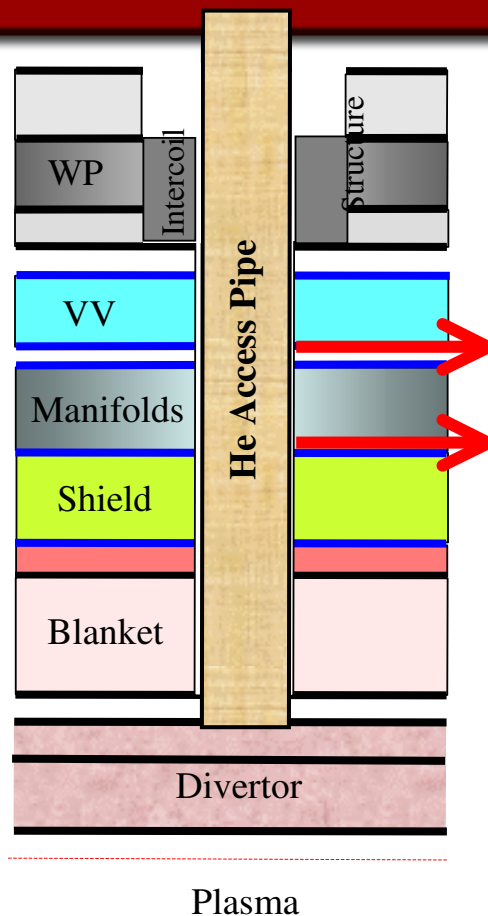
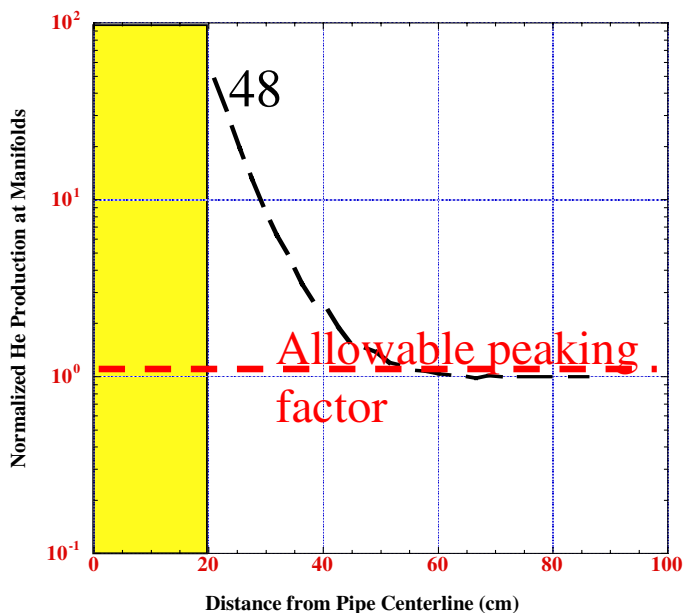
Nominal
value: 135
dpa

Design
Limit: 200
dpa

Shield surrounding pipe is not life-of-plant component



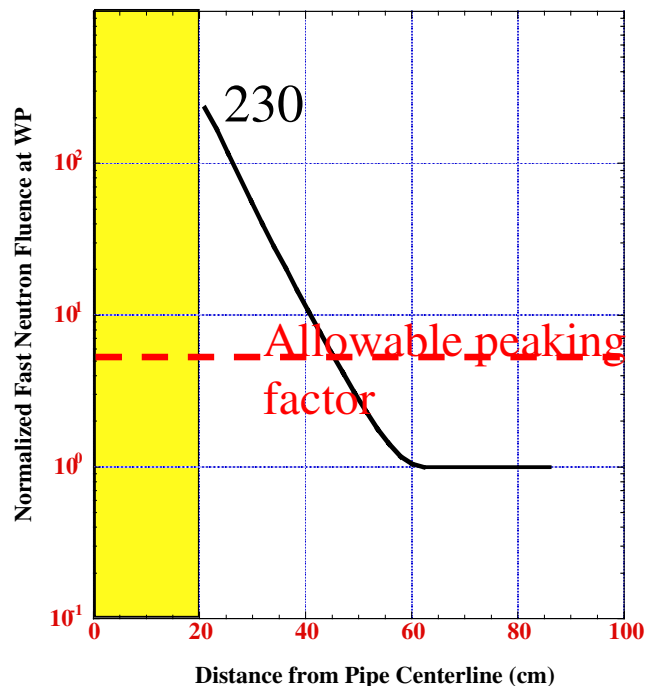
2-D Results (Manifold and Vacuum Vessel)



Parts of manifolds and vacuum vessel surrounding pipe cannot be rewelded

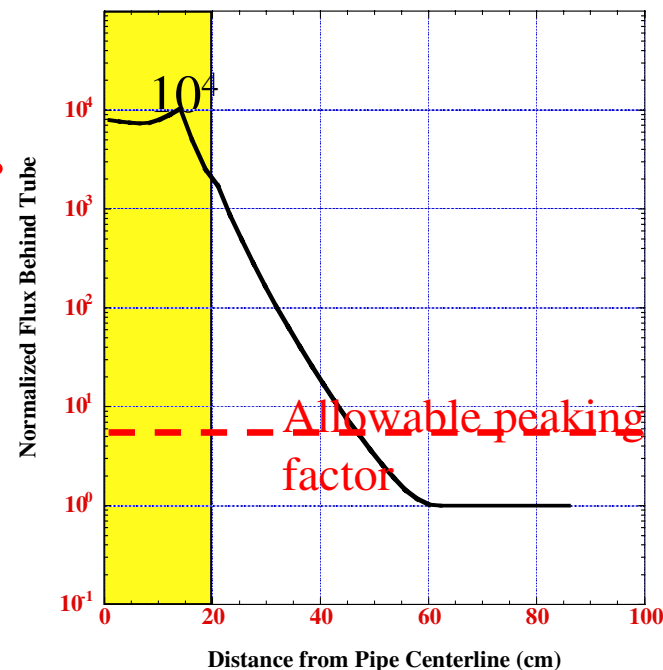
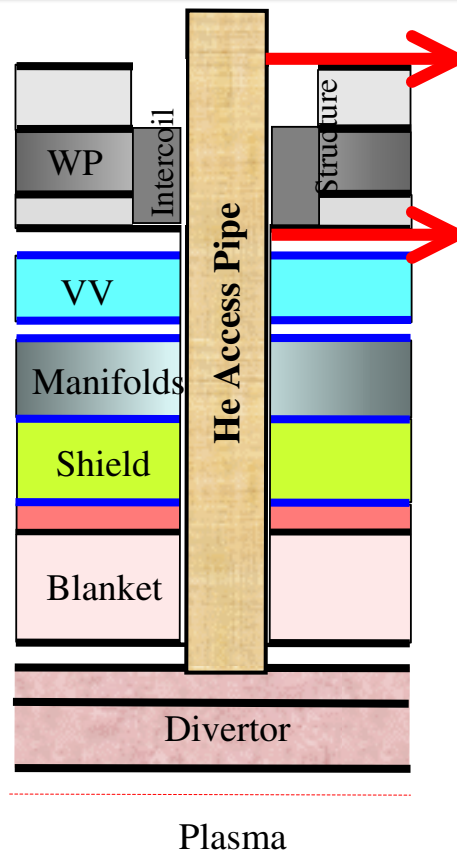


2-D results (Magnet)



Nominal value: $0.2 \times 10^{19} \text{ n/cm}^2$

Design Limit: 10^{19} n/cm^2



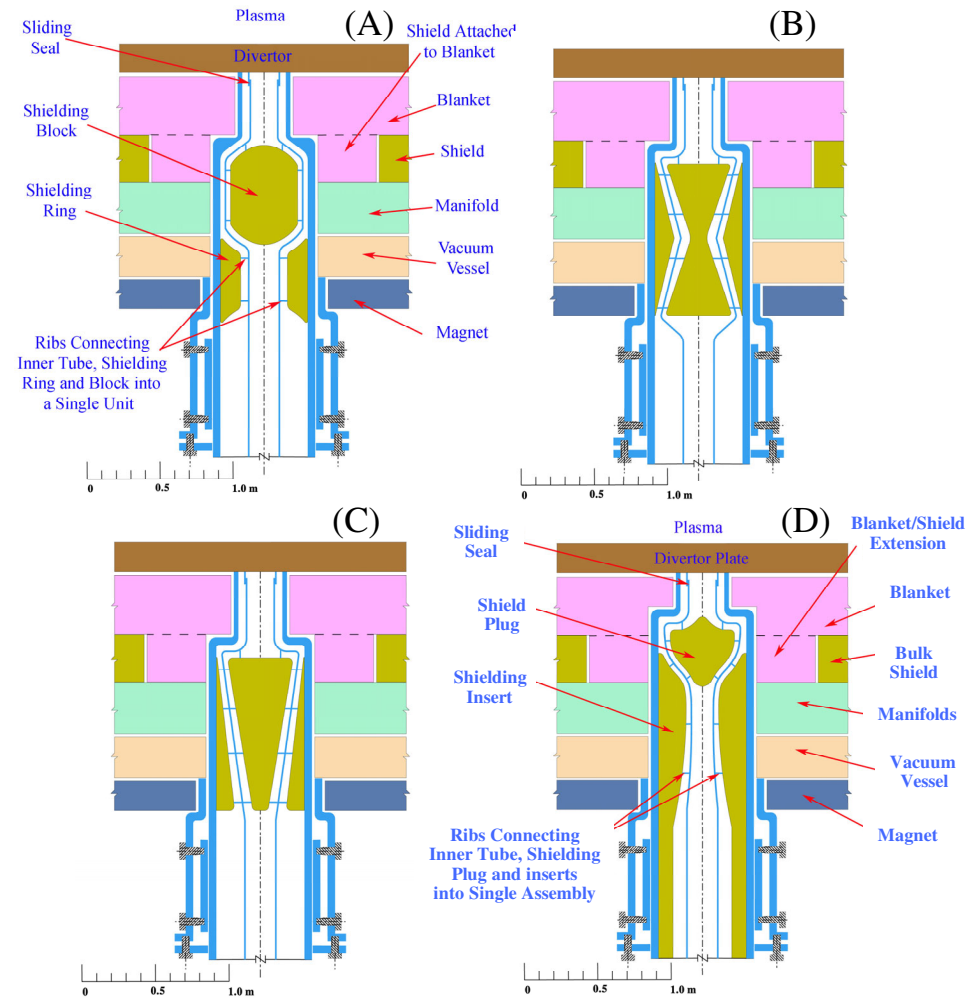
Nominal value: $10^7 \text{ n/cm}^2 \cdot \text{s}$

Design Limit: $5 \times 10^7 \text{ n/cm}^2 \cdot \text{s}$

- Winding pack should be placed 30-40 cm from pipe
- Flux at end of pipe is excessive $\Rightarrow$ protect externals with local shields.



3-D Model





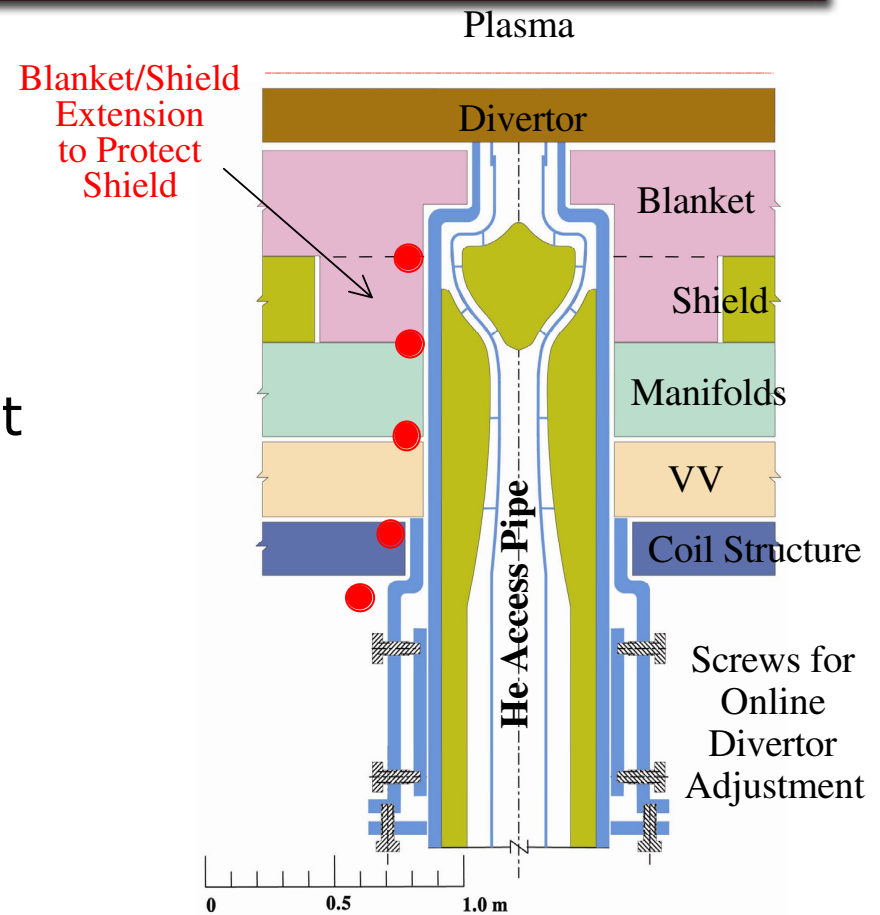
3-D Neutronics Model

- **Purpose:**

- Examine effectiveness of shield plug and inserts
- Estimate peaking in damage due to streaming
- Assess radiation environment behind magnet due to streaming

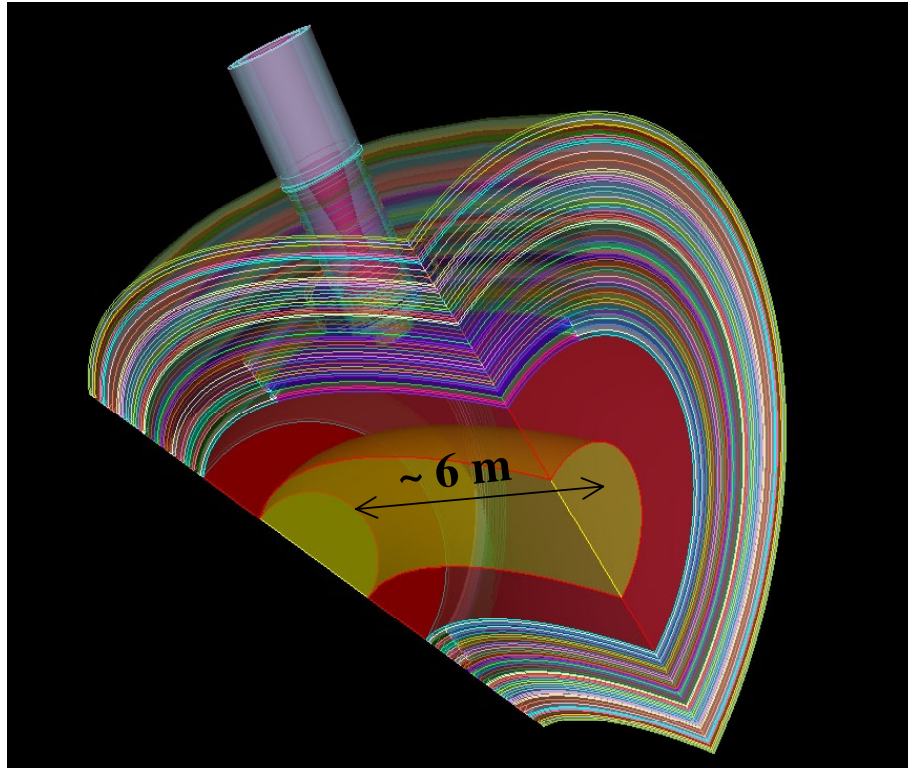
- **3-D codes:**

- Attila: discrete ordinates
- DAG-MCNPX: Monte Carlo

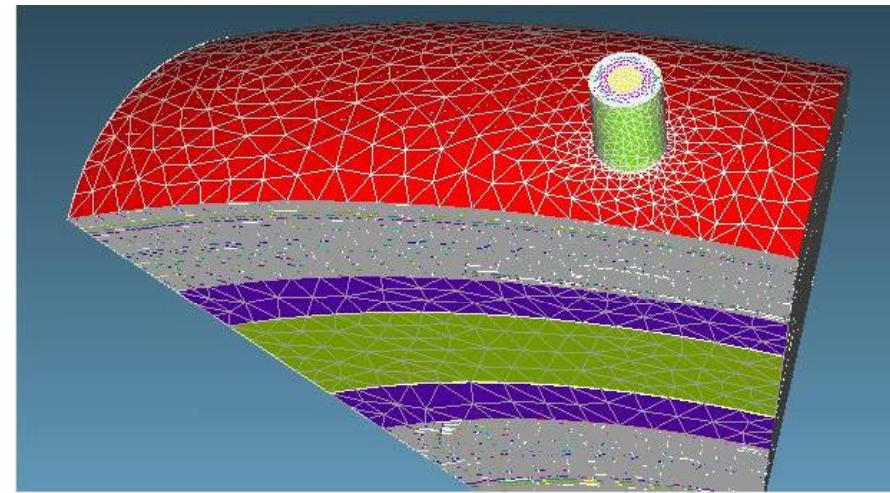




3D CAD Models



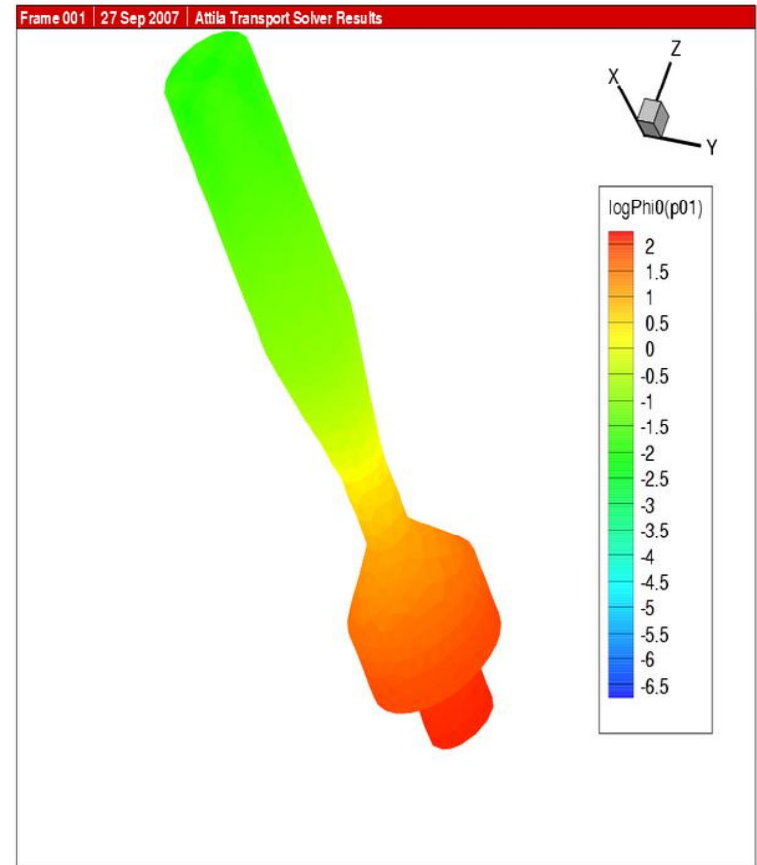
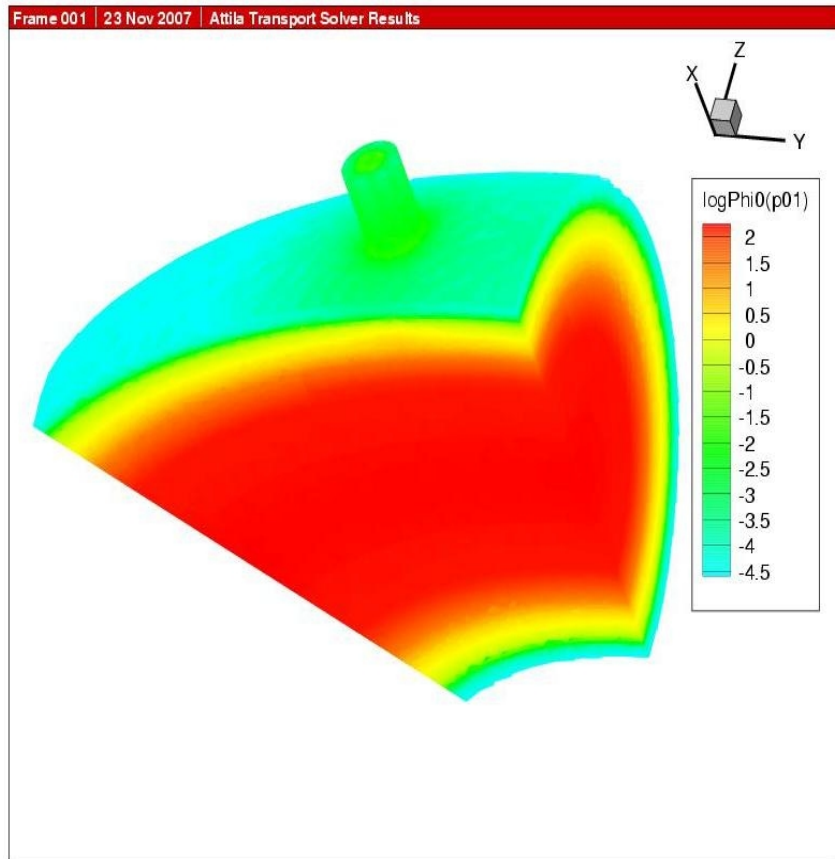
CAD model



Tetrahedral mesh
representation (Attila)



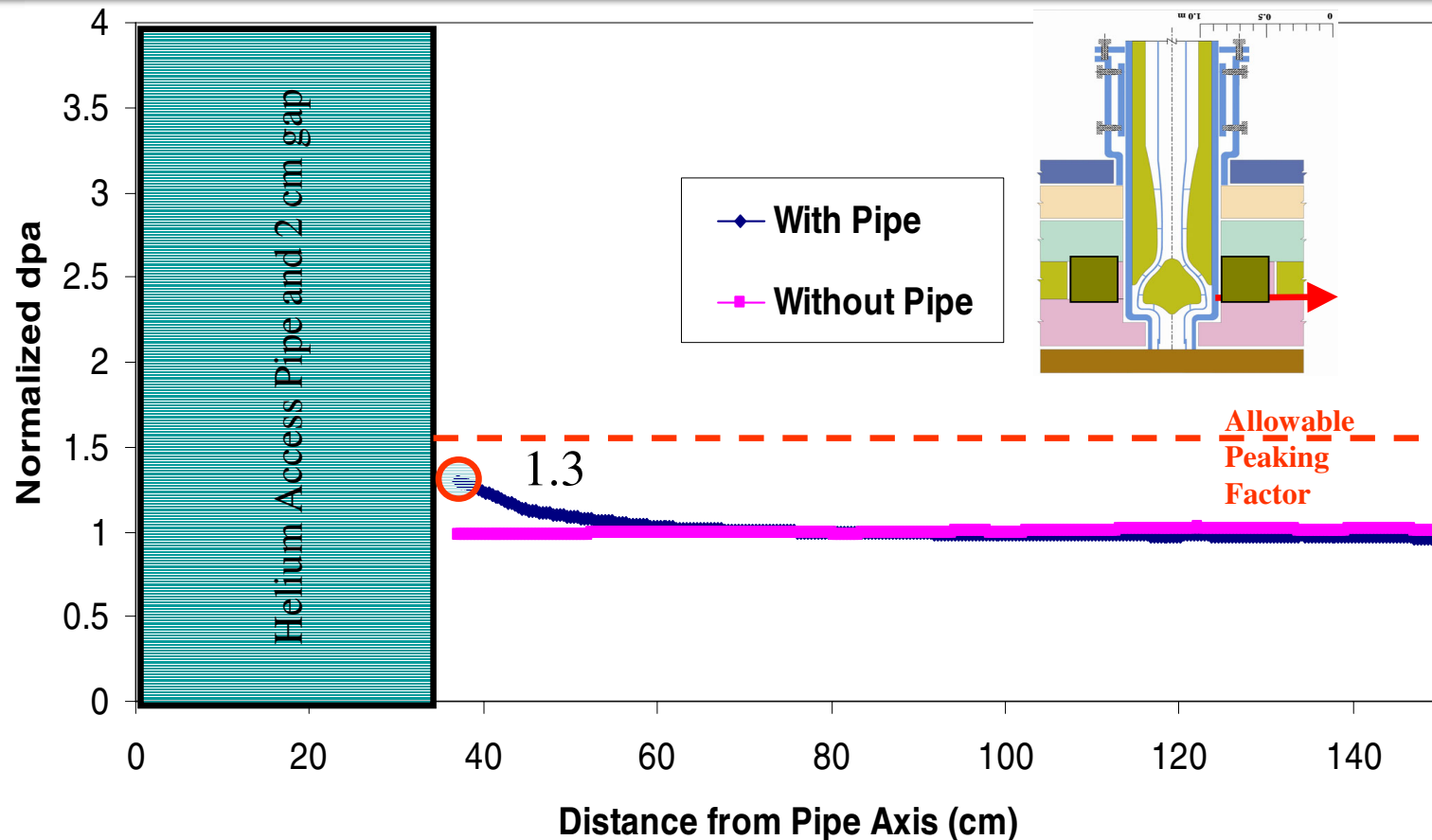
3-D Results: Neutron Flux Map



- Streaming causes 3-4 orders of magnitude increase in flux behind magnet.
- Shielding plug attenuates flux by a factor of 20.



3-D Results: Shield



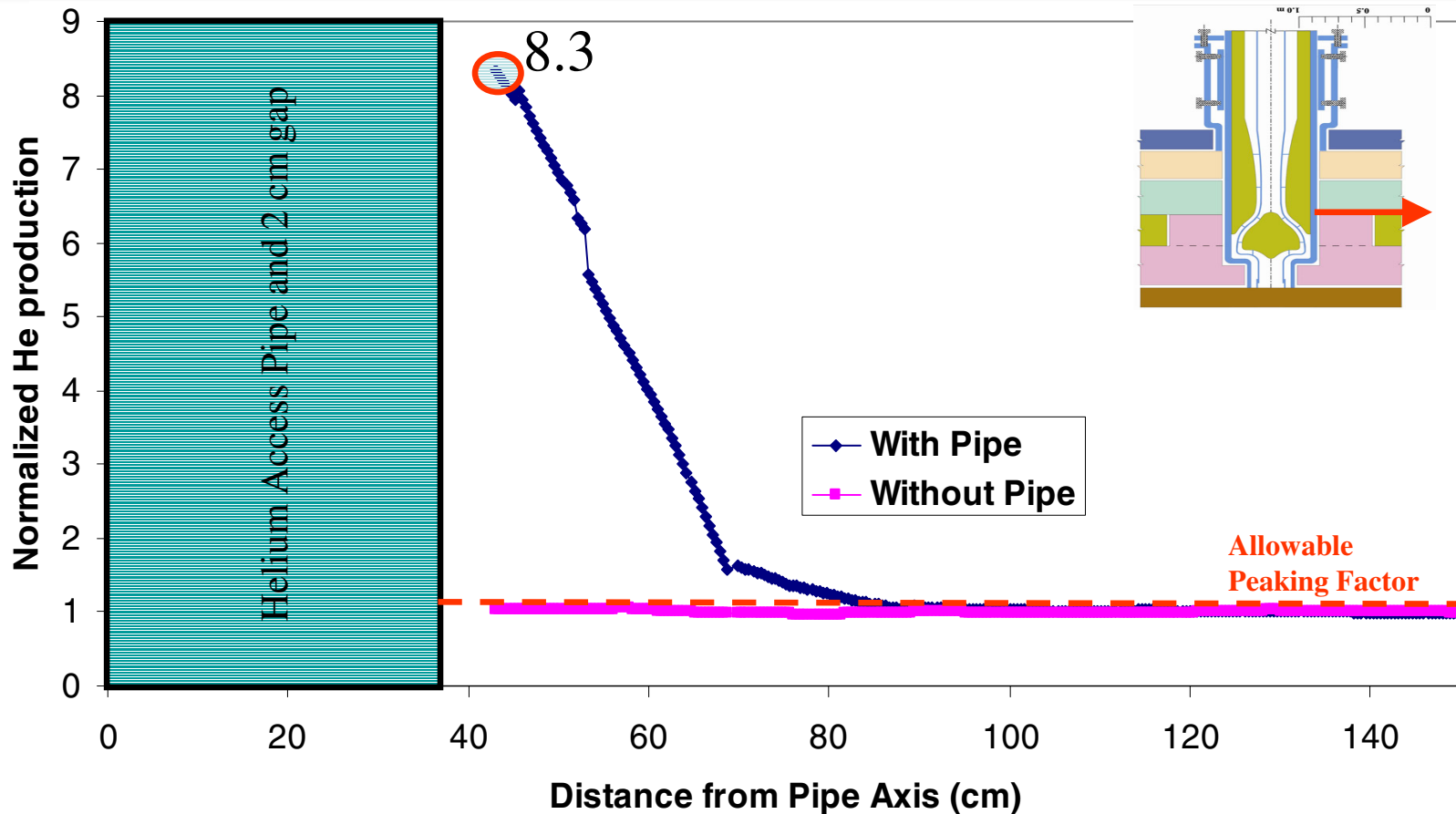
Peaking Factor = 1.3 $\Rightarrow$ $\sim$ (< 200 dpa limit)

$\Rightarrow$ Bulk shield is permanent component

$\Rightarrow$ No need for blanket/shield extension.



3-D results: Manifold

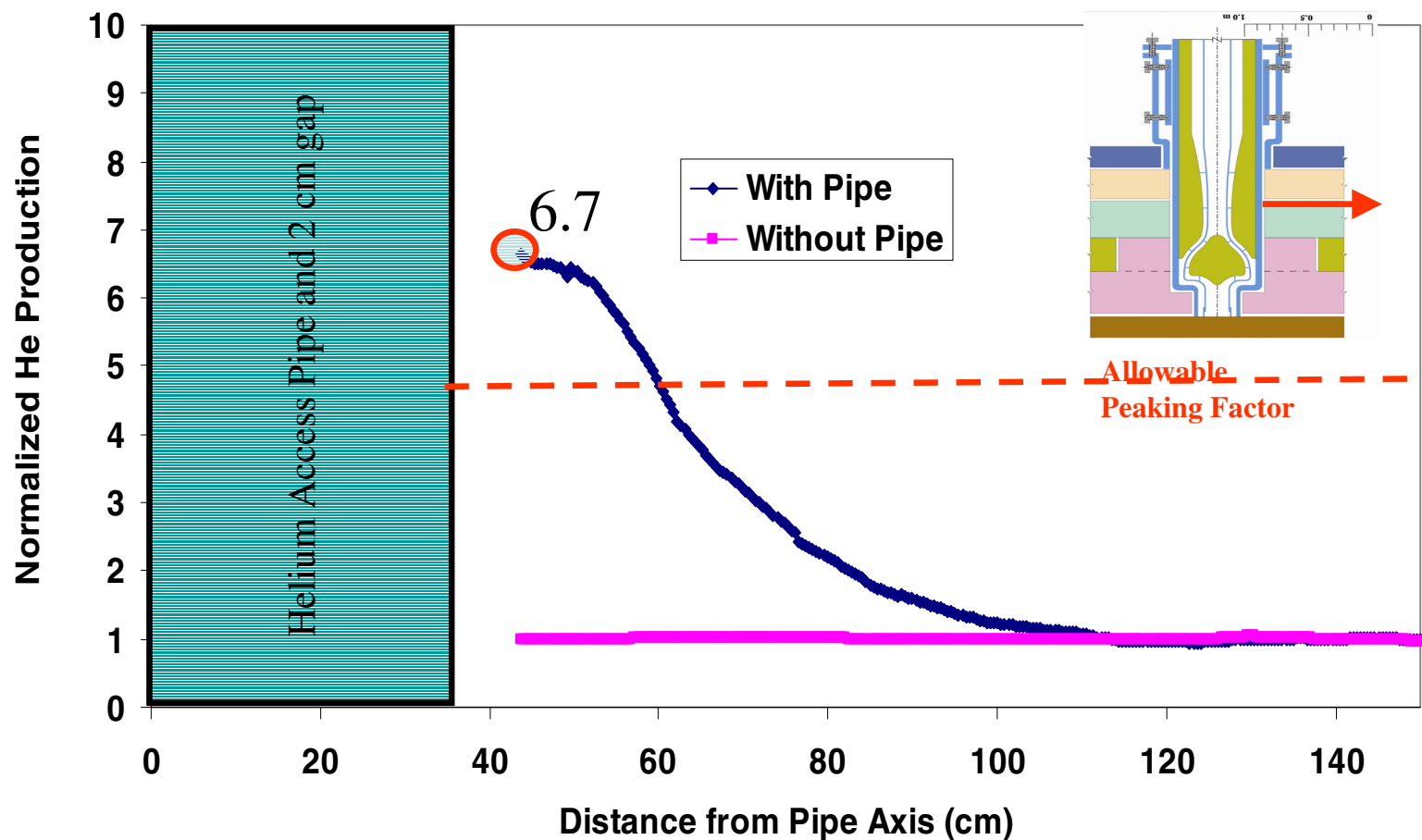


Peaking Factor = 8.3 $\Rightarrow$ ~ 8 He appm @ 40 FPY (> 1 appm limit)

$\Rightarrow$ Avoid rewelding 40 cm thick manifolds surrounding pipe



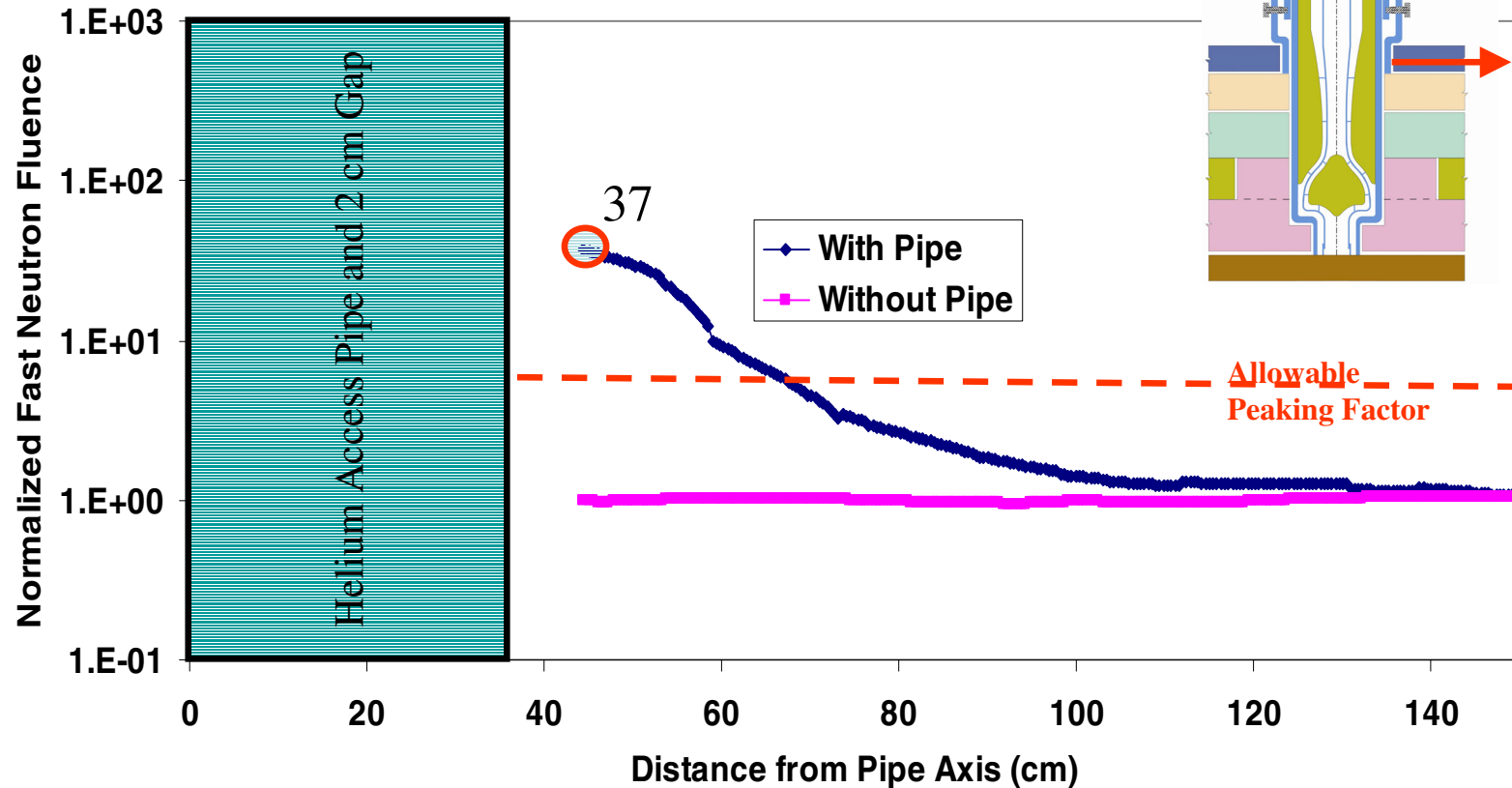
3-D Results: VV



Peaking Factor = 6.7 $\Rightarrow$ ~ 1.3 He appm @ 40 FPY (> 1 appm limit)
 $\Rightarrow$ Avoid rewelding 20 cm thick VV surrounding pipe



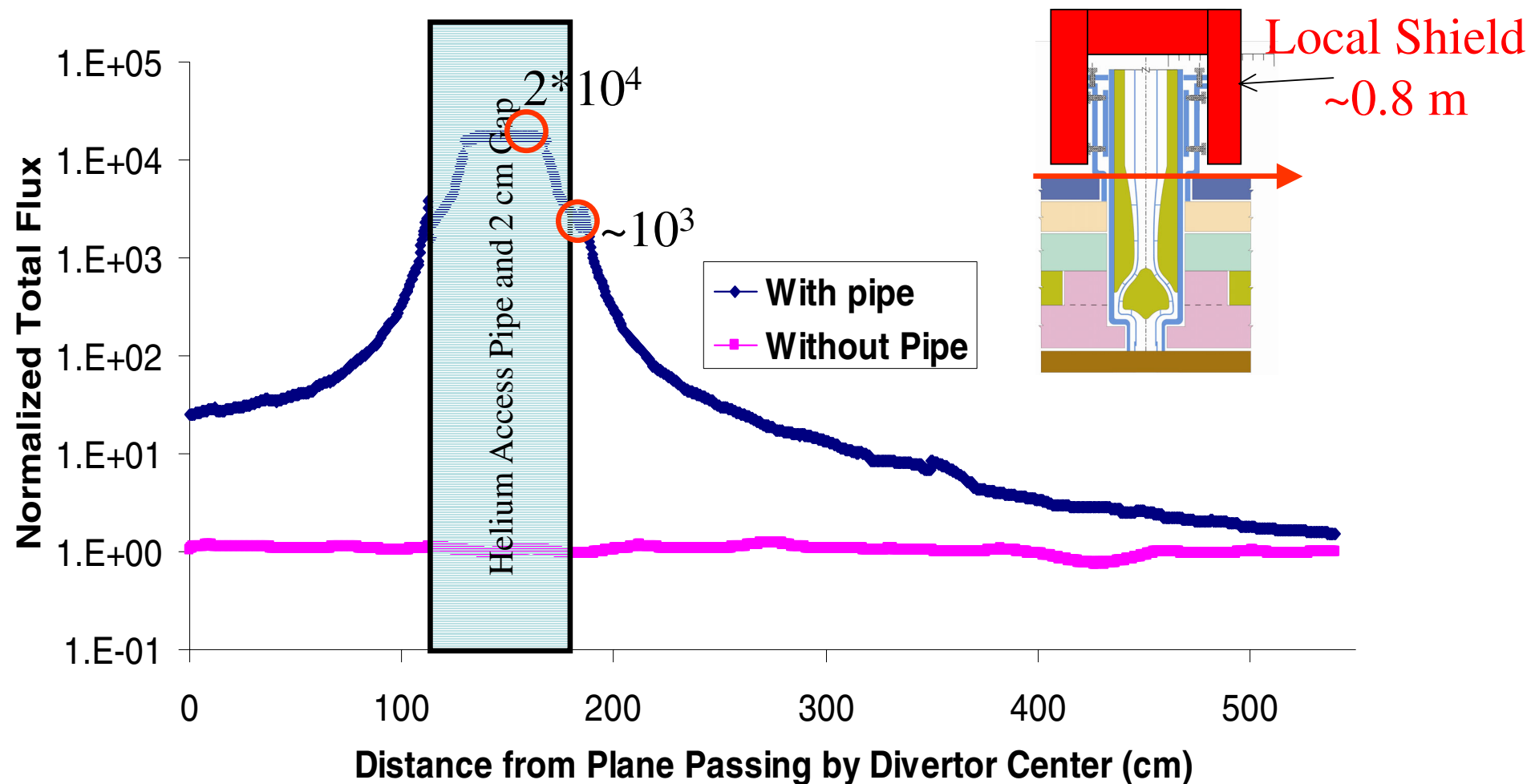
3-D Results: Magnet



Peaking Factor = 37 $\Rightarrow \sim 9 \times 10^{19} \text{ n/cm}^2 \text{ @ } 40 \text{ FPY}$ ($> 10^{19} \text{ n/cm}^2$ limit)
 $\Rightarrow$ Winding pack should be placed 40 cm away from pipe

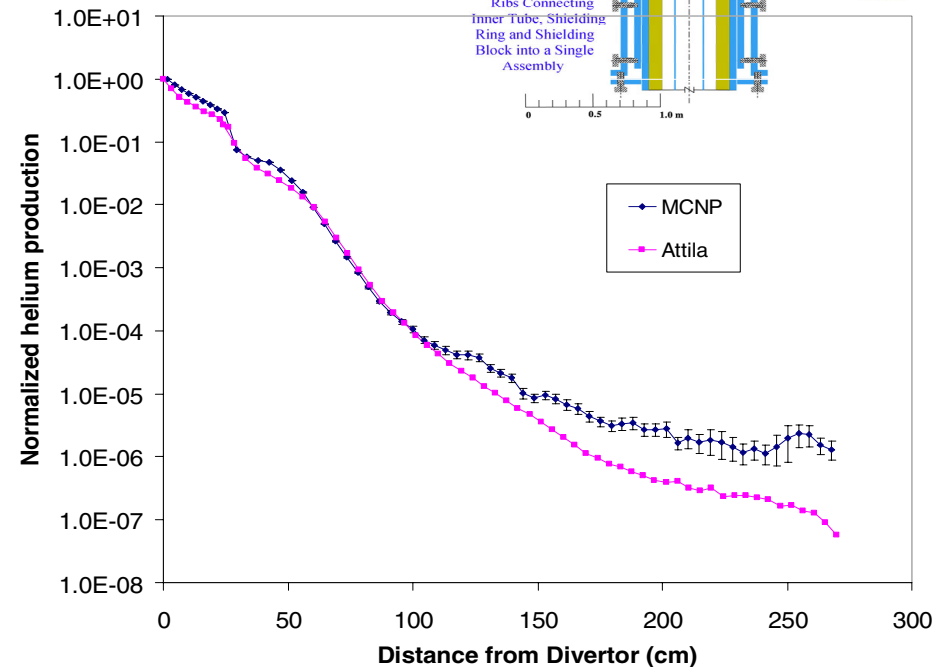
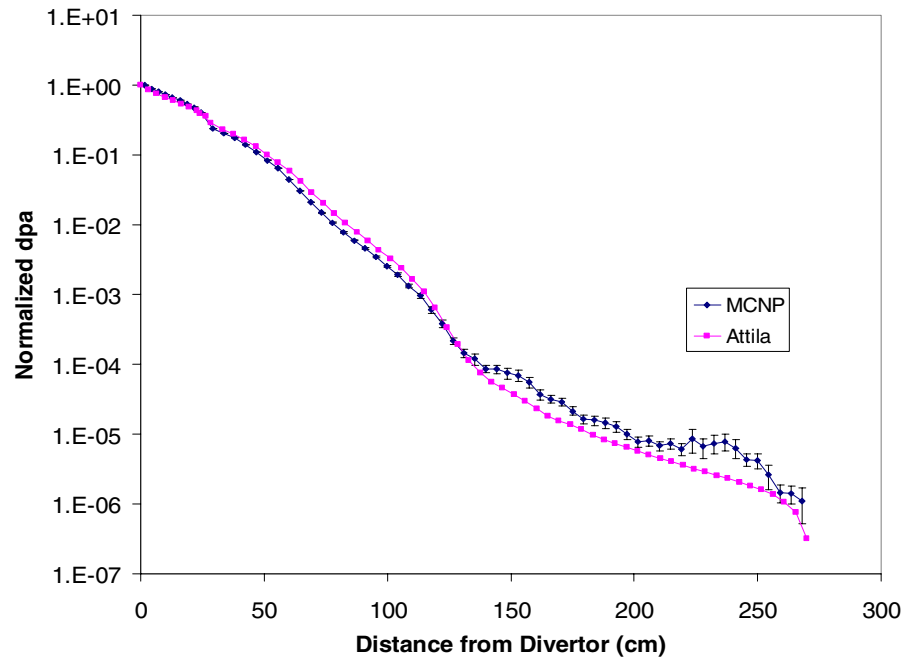
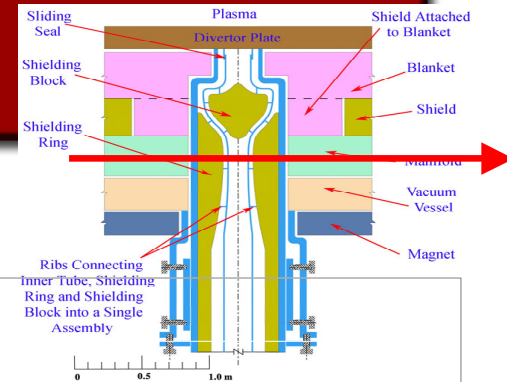


3-D Results: Magnet





3-D Results: Pipe



- Pipe wall lifetime ~ 4.5 FPY, exceeding divertor lifetime (3 FPY)
- Front 60 cm of pipe wall is not reweldable after 3 FPY and should be replaced with divertor
- Damage to screws (at ~2 m) is negligible



Conclusions

- **3-D results indicate:**
 - No need for blanket/shield extension.
 - Helium production at manifolds and VV exceeds reweldability limit by 2-8 fold
 - ⇒ avoid rewelding within 20-40 cm from pipe.
 - Winding pack should be placed at least 40 cm from pipe.
 - Front 60-120 cm of pipe wall is not reweldable and should be replaced along with divertor system every 3 FPY.
 - Neutron-induced swelling in screws is negligible ($< 0.05\%$).
- Neutron flux behind pipe is excessive, calling for 60-80 cm thick local shield to protect externals.
- Neutron attenuation through shielding plug and inserts is not sufficient to eliminate streaming problems entirely.



Recommendations

- Future studies could develop more effective scheme(s) to attenuate streaming neutrons and reduce flux outside pipes. For example, simple pipe with smaller diameter (< 60 cm) and several right-angle bends represents a better approach, eliminating the need for massive shielding plug and inserts (170 tons for 24 pipes of ARIES-CS).



Produced by University Communications

Questions

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