

Status of 3-D Analysis, Neutron Streaming through Penetrations, and LOCA/LOFA Analysis

L. El-Guebaly,

M. Sawan, P. Wilson, D. Henderson, A. Ibrahim, G. Sviatoslavsky, B. Kiedrowski, T. Tautges, C. Martin

Fusion Technology Institute UW - Madison

Contributors: X. Wang, S. Malang (UCSD)

ARIES-CS Project Meeting

October 4 - 5, 2006 PPPL



Final Radial Build

(<u>http://fti.neep.wisc.edu/aries-cs/builds/build.html</u>) (5.3 MW/m² Peak NWL)





Status of 3-D TBR Analysis

Purpose: compute overall TBR and M_n

CAD model received from UCSD on 9/19/06 including:

FW/Blanket/BW Shield Manifolds VV Magnet Divertor No penetrations!



Essential components for TBR and M_n will be included in 3-D model: FW/Blanket/BW Shield Manifolds (continuous) Divertor ECH ducts (to be added by UW).



Status of 3-D TBR Analysis (Cont.)

- Complex 3-D problem, requiring many time-consuming iterations.
- UW had difficult time transferring UCSD model into ACIS.
- UW manipulated UCSD geometry to merge properly even though Xueren tried improving accuracy settings in Pro/E.
- Clean model shows no sign of overlapping. **3-D analysis in progress**.
- Future ARIES modeling work should be produced in ACIS*.



^{*} Commercial solid modeling engine underlying CGM and CUBIT, analogous to what's under Pro/E.



Neutron Streaming through Penetrations

• Neutron streaming through penetrations compromises shielding performance.

• ARIES-CS 7 types of penetrations:

- 198 He tubes for blanket (32 cm ID)
- 24 Divertor He access pipes (~30 cm ID)
- 30 Divertor pumping ducts (42 x 120 cm each)
- 12 Large pumping ducts (1 x 1.25 m each)
- 3 ECH ducts (24 x 54 cm each).
- 6 main He pipes HX to/from blanket (72 cm ID each)
- 6 main He pipes HX to/from divertor (70 cm ID each)

Potential solutions:

- Local shield behind penetrations
- He tube axis oriented toward lower neutron source
- Penetration shield surrounding ducts
- Replaceable shield close to penetrations
- Rewelding of VV and manifolds avoided close to penetrations
- Bends included in some penetrations.











Streaming through Blanket He Tubes (9/15/05 Presentation)

- 198 He tubes (30 cm ID) connect manifolds to blanket modules.
- 2-D results:
 - High damage at tube wall
 - High damage at VV and magnet behind tubes.
- Solutions:
 - Orient tube toward lower n source
 - Avoid welding near tube wall
 - Replaceable shield close to tube
 - Local shield behind tube to protect VV and magnet.







Streaming through **Divertor System**





Streaming through Divertor Pumping Ducts





Effect of Streaming through Divertor He Access Pipes

NWL near pipes= $0.5 - 1 \text{ MW/m}^2$.

2-D model of pipe estimated:

- dpa at shield
- He production at manifolds
- He production at VV
- Fast neutron fluence at WP
- Flux behind pipe
- dpa & He production along pipe wall.

3-D streaming analysis is underway using Attila code.



Plasma



Part of Shield Surrounding Pipe Should be Replaceable





Part of Manifolds Surrounding Pipe Cannot be Rewelded





Part of VV Surrounding Pipe Cannot be Rewelded





30-40 cm Inter-coil Structure around Pipe Protects Sides of WP





High Flux Behind Pipe Calls for Local Shield to Protect Externals





Pipe Wall



Front part of pipe wall should be replaceable

Pipe wall is not reweldable \Rightarrow Replace pipe with divertor

200



Recommendations



ZZZ Replaceable Shield

Not Reweldable





Ongoing 3-D Streaming Analysis Using Attila Code

Purpose: Determine dimensions of replaceable shield, non-reweldable manifolds and VV, and local shield.



QuickTime™ and a TIFF (Uncompressed) decompressor are needed to see this picture.



Alternate Pipe Design (S. Malang)





Comments on Alternate Pipe Design

- Shielding ring and block do not solved streaming problem entirely:
 - 10 cm thick WC **shielding ring** is insufficient to protect pipe wall, bulk shield, and manifolds
 - \Rightarrow Parts around pipe should be replaceable and cannot be rewelded.
 - Shielding block will not attenuate flux behind pipes by 4 orders of magnitude
 - \Rightarrow Local shield behind pipe is still needed.
- Sizable WC shield added:

Mass / pipe (ton)	Alternate Design	Baseline Design
Shielding Ring	~1.6	
Shielding Block	~1	
2.5 cm FS Pipe Wa	ull 0.6	0.4
Total mass - 24 pipes	~77 ton	~10 ton





LOCA / LOFA Analysis (2.6 MW/m² Ave. NWL; 0.5 MW/m² Surface Heat Load)

Initial temperatures at onset of LOCA/LOFA:

FW	500 °C	Shield & Manifolds	450 °C
SiC	550 °C	VV	100 °C
Back wall	450 °C	LiPb average temp	625 °C

Assumptions: Plasma on for 3 s; Instantaneous loss of coolant.





Uniform Blanket and Shield (He & LiPb LOCA in <u>all</u> Modules; H₂O LOFA in VV)

 $\varepsilon = 0.3$ all surfaces

 $\epsilon = 0.5$ for SiC



Max temp = 753 °C

Max temp = 700 °C



Non-uniform Blanket and Shield (He & LiPb LOCA in <u>all</u> Modules; H₂O LOFA in VV)

$\varepsilon = 0.3$ all surfaces

 $\epsilon = 0.5$ for SiC



Max temp = 762 °C

Max temp = 720 °C



Summary and Conclusions

- 3-D TBR and streaming analyses are underway.
- No shielding ring or block should be included in divertor He access pipes.
- Ongoing 3-D streaming analysis will determine size of replaceable shield and non-reweldble manifolds/VV around pipes.
- More realistic LOCA/LOFA assumptions help reduce temperature during accident.