

Radial Build Definition for LiPb/FS and Li/FS Systems

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http://fti.neep.wisc.edu/FTI/ARIES/SEP2003/lae_radialbuild.pdf

ARIES-CS Project Meeting
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Candidate Blanket Concepts

<u>Breeder</u>	<u>Multiplier</u>	<u>Structure</u>	<u>Blanket/Shield Coolant</u>	<u>VV Coolant</u>	<u>VV Location</u>
<u>ARIES-CS:</u>					
Flibe	Be	FS	Flibe	H ₂ O	Inside Magnet
LiPb	—	SiC	LiPb	H ₂ O	Inside Magnet
LiPb	—	FS	He	H ₂ O	Inside Magnet
LiPb	—	FS	He	He	Outside Magnet
Li	—	FS	He/Li	He	Outside Magnet
<u>SPPS:</u>					
Li	—	V	Li	He	Outside Magnet

Initial Parameters

(1/31/03 Strawman - L.P. Ku)

Net Electric Power	1000 MW _e
Fusion Power	2000 MW
# of Field Periods	3
A	4.5
< R >	8.25 m
< a >	1.85 m
Average Neutron Wall Loading (Γ)*	2 MW/m ²
FW Area	~ 800 m ²
Toroidal Length of Field Period	~ 9 m
Minimum Plasma to Mid-Coil Distance (Δ_{\min})	1.2 m
# of Δ_{\min} per Field Period	2
FW Coverage Fraction[#] for Shield-only Zones	8%

* Peak Γ is ~3 MW/m².

Sensitive to difference between Δ_{\min} and nominal Δ .

Design Requirements and Radiation Limits

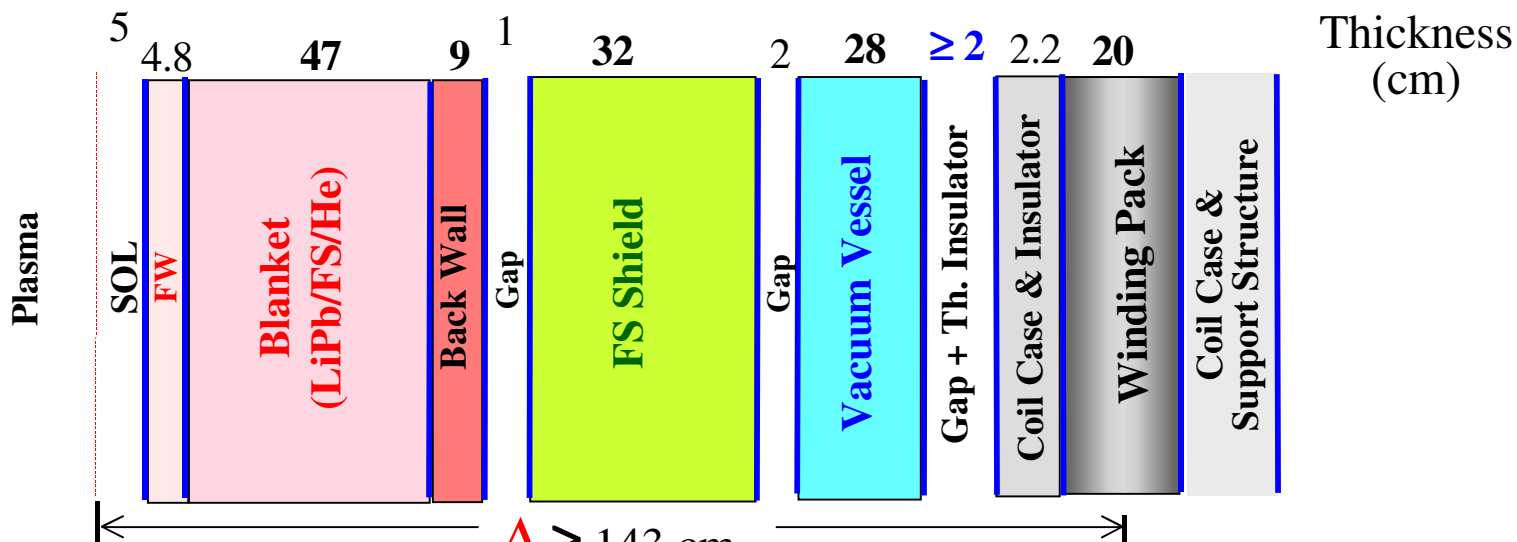
Overall TBR (for T self-sufficiency)	1.1	
dpa to FS Structure (for structural integrity)	200	dpa
Helium Production @ VV (for reweldability of FS)	1	appm
LT S/C Magnet (@ 4K):		
n fluence to Nb ₃ Sn ($E_n > 0.1$ MeV)	10 ¹⁹	n/cm ²
Dose to polyimide insulator	10 ¹¹	rads
Nuclear heating	2	mW/cm ³
dpa to Cu stabilizer	6x10 ⁻³	dpa
Biological dose outside building (dose during operation for workers and public protection)	2.5	mrem/h

Assumptions

- Peak to average $\Gamma = 1.5$
- RF Penetrations and assembly gaps occupy 2% of FW area.
- Divertor plates/baffles:
 - 5 cm thick
 - 50% structure and 50% He coolant
 - Cover 15% of FW area.
- SPSS magnet composition.
- 8% FW coverage fraction of shield-only zones used for all blanket concepts.
- 1-D poloidal neutronics model with av. $a = 1.85$ m.

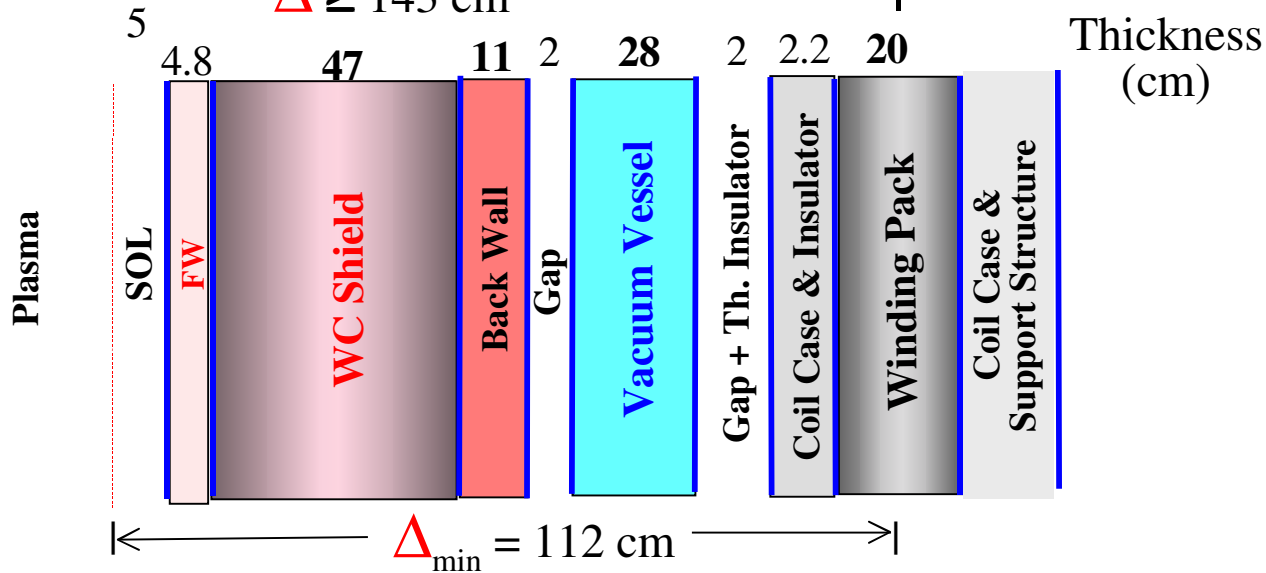
Recommended LiPb/FS/He Radial Build (water-cooled internal VV)

**Blanket
Zones**



**Shield
Only
Zones**

Boundary between WC-shield and back wall will be adjusted to meet design requirements



LiPb/FS/He System

(water-cooled internal VV)

Components

FW

Blanket

WC Shield*

FS Shield

VV

Winding Pack

(Composition not available. Used SPPS')

Composition

31% FS Structure
69% He Coolant

90% LiPb with 90% enriched Li
3% FS Structure
7% He Coolant

90% WC Filler
3% FS Structure
7% He Coolant

15% FS Structure
10% He Coolant
75% Borated Steel Filler

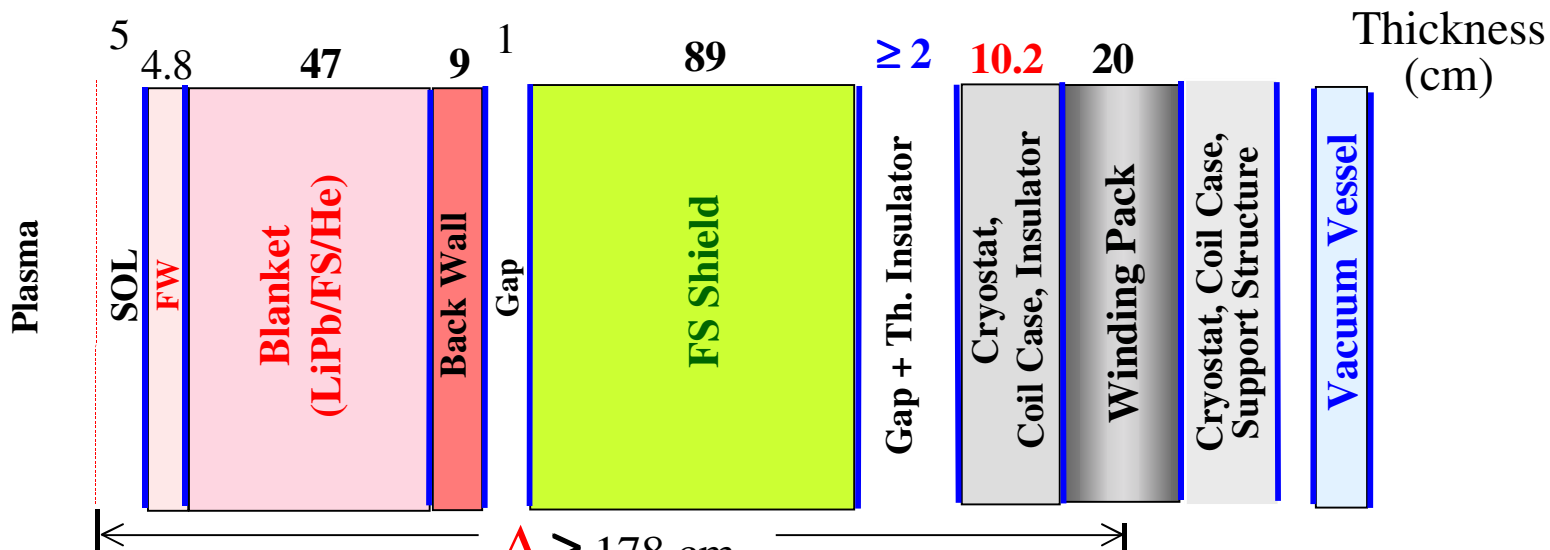
28% FS Structure
49% Water
23% Borated Steel Filler

25% Incoloy Structure
20% Cu Stabilizer
15% Nb₃Sn + Conduit
25% GFF Polyimide
15% LHe

* Same constituents for blanket with WC replacing LiPb.

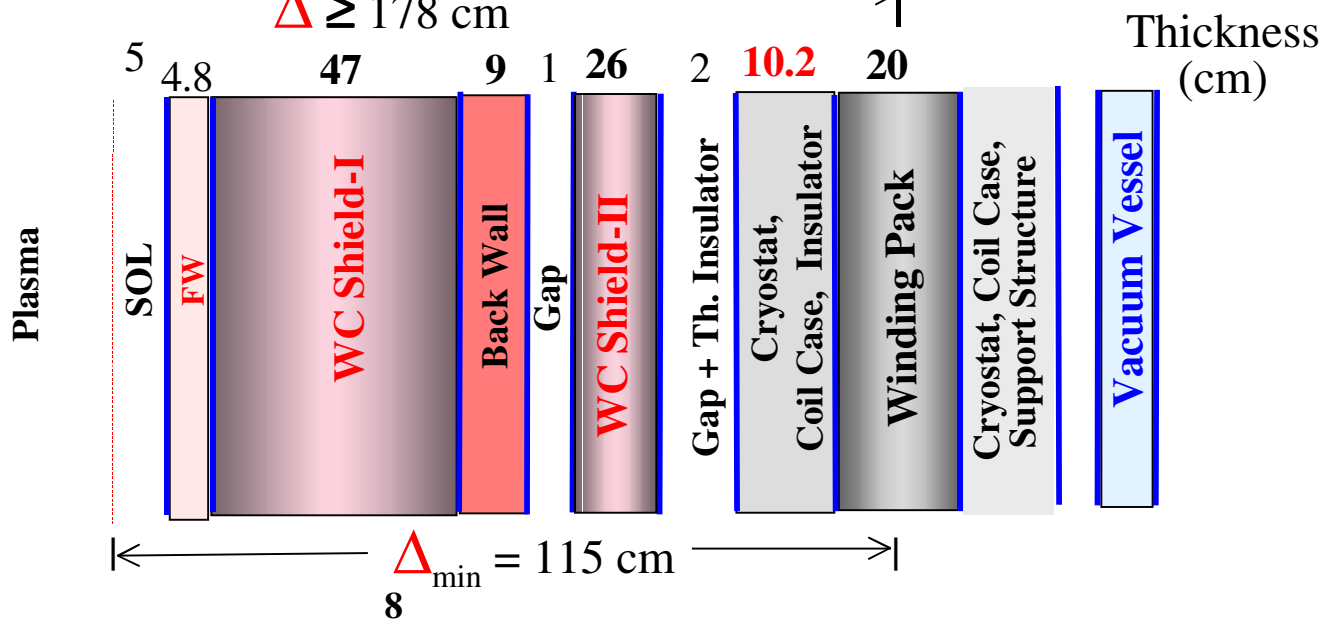
Recommended LiPb/FS/He Radial Build (He-cooled External VV)

**Blanket
Zones**



**Shield
Only
Zones**

Boundary between WC-shield and back wall will be adjusted to meet design requirements



LiPb/FS/He System (He-cooled External VV)

Components

FW

Blanket

WC Shield-I*

WC Shield-II#

FS Shield

Winding Pack

(Composition not available. Used SPPS*)

VV

Composition

31% FS Structure
69% He Coolant

90% LiPb with 90% enriched Li
3% FS Structure
7% He Coolant

90% WC Filler
3% FS Structure
7% He Coolant

15% FS Structure
10% He Coolant
75% WC Filler

15% FS Structure
10% He Coolant
75% Borated Steel Filler

25% Incoloy Structure
20% Cu Stabilizer
15% Nb₂Sn + Conduit
25% GFF Polyimide
15% LHe

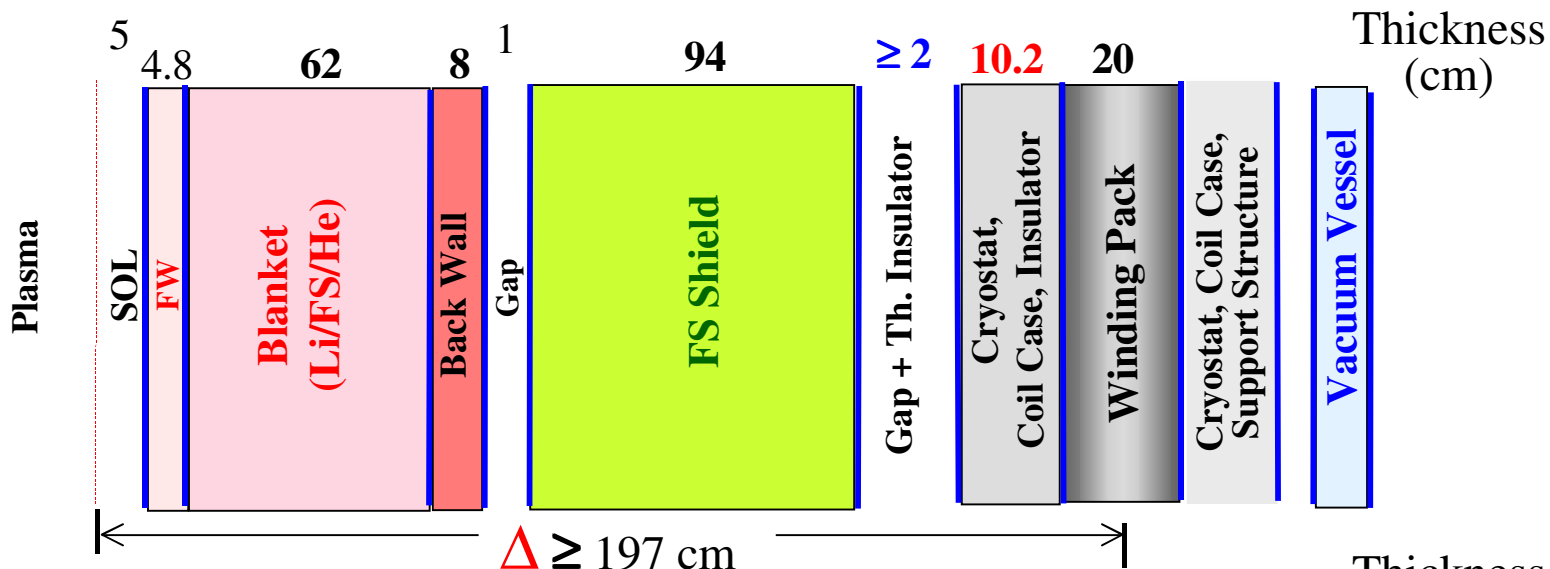
30% FS Structure
70% He

* Same constituents for blanket with WC replacing LiPb.

Same constituents for FS-shield with WC replacing B-FS filler. ZrH_{1.7} filler could save 2-3 cm in radial build.

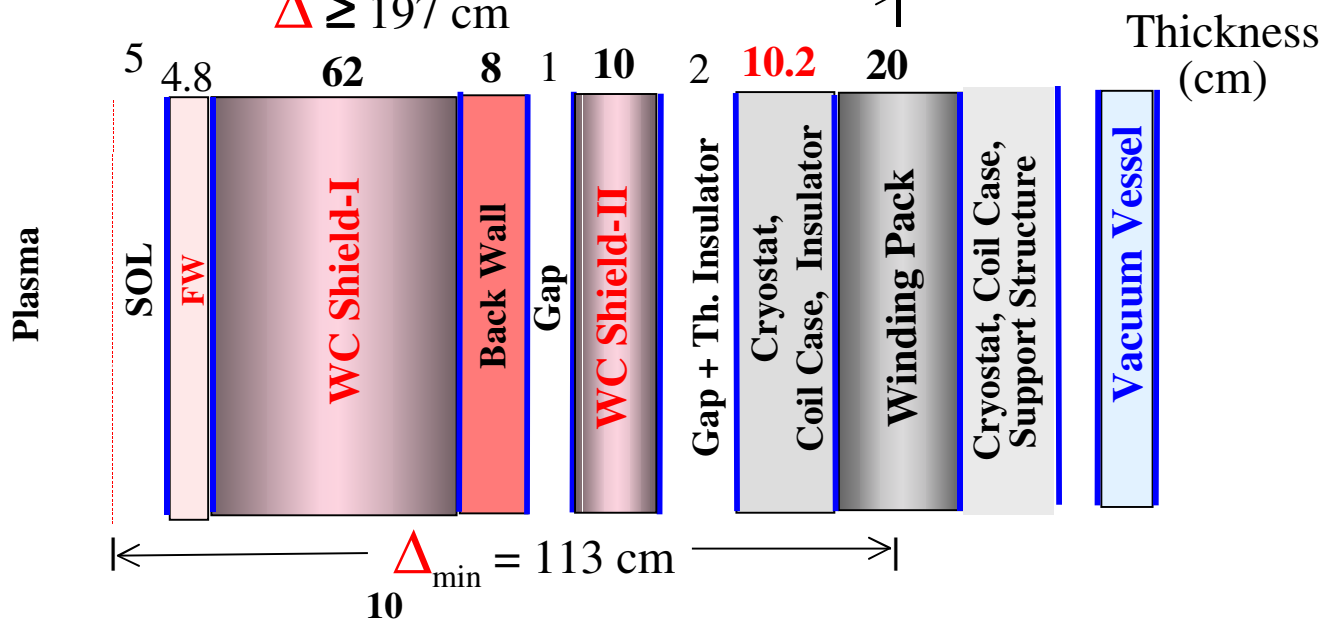
Recommended Li/FS/He Radial Build (He-cooled External VV)

**Blanket
Zones**



**Shield
Only
Zones**

Boundary between WC-shield and back wall will be adjusted to meet design requirements



Li/FS/He System

(He-cooled External VV)

Components

FW

Blanket

WC Shield-I*

WC Shield-II#

FS Shield

Winding Pack

(Composition not available. Used SPSS')

VV

Composition

31% FS Structure
69% He Coolant

90% Li (natural)
3% FS Structure
7% He Coolant

90% WC Filler
3% FS Structure
7% He Coolant

15% FS Structure
10% He Coolant
75% WC Filler

15% FS Structure
10% He Coolant
75% Borated Steel Filler

25% Incoloy Structure
20% Cu Stabilizer
15% Nb₃Sn + Conduit
25% GFF Polyimide
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30% FS Structure
70% He

* Same constituents for blanket with WC replacing LiPb.

Same constituents for FS-shield with WC replacing B-FS filler. ZrH_{1.7} filler could save 2-3 cm in radial build.

Nominal Blanket and Shielding Components

Blanket Concept	ARIES-CS					SPPS
	Internal VV			External VV		External VV
	<u>Flibe/FS/Be</u>	<u>LiPb/SiC</u>	<u>LiPb/FS/He</u>	<u>LiPb/FS/He</u>	<u>Li/FS/He</u>	<u>Li/V</u>
<u>Thickness (cm):</u>						
FW/Blanket-I	33	25	52	52	67	36
Gap	—	1	—	—	—	—
FW/Blanket-II	—	25	—	—	—	—
FS Back Wall	—	—	9	9	8	—
Gap	1	—	1	1	1	2
HT Shield	45	35	32	89	94	45
Gap	2	2	2	—	—	2
LT Shield	—	—	—	—	—	35
VV	25	25	28	—	—	—
Total B/S/VV	106	113	124	151	170	120
Net Change (compared to SPPS)	- 14	- 7	+ 4	+ 31	+ 50	—

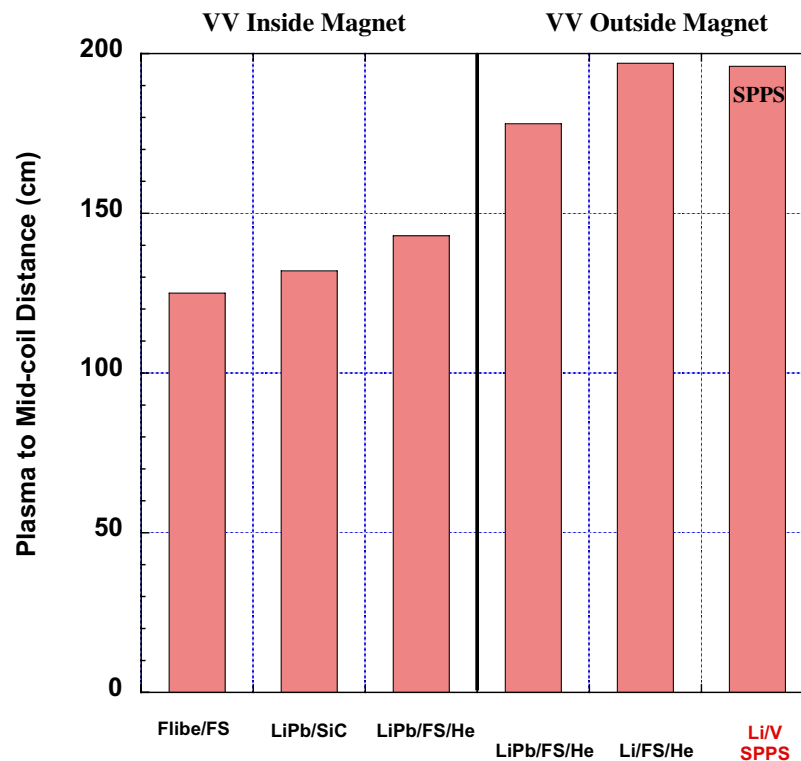
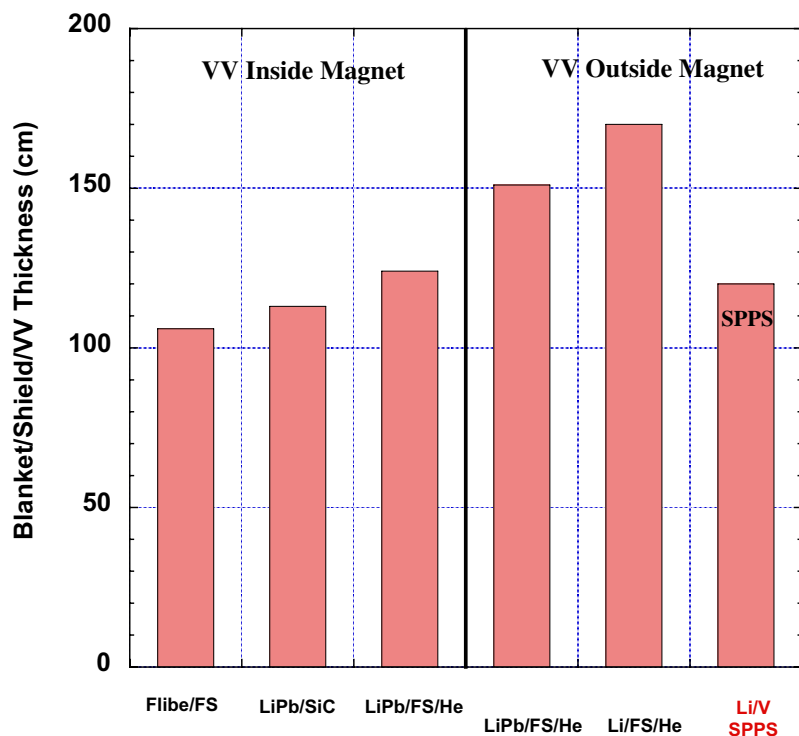
Nominal Plasma to Mid-Coil Distance

Blanket Concept	ARIES-CS					SPPS
	Internal VV			External VV		External VV
	<u>Flibe/FS/Be</u>	<u>LiPb/SiC</u>	<u>LiPb/FS/He</u>	<u>LiPb/FS/He</u>	<u>Li/FS/He</u>	<u>Li/V</u>
<u>Thickness (cm):</u>						
SOL	5	5	5	5	5	15
FW/B/S/VV	106	113	124	151	170	120
Gap + Th. Insl.	≥ 2	≥ 2	≥ 2	≥ 2	≥ 2	≥ 8
Cryostat*	2.2	2.2	2.2	10.2	10.2	15
1/2 Winding Pack	10	10	10	10	10	38
Δ	125	132	143	178	197	196
Net Change (compared to SPPS)	- 71	- 64	- 53	- 18	+ 1	-

* Includes coil case and electric insulator.

Flibe system offers most compact radial build

Comparison Between Nominal Radial Builds



Water-cooled internal VV helps reduce nominal radial build

Recommended Shield-Only Zones for Δ_{\min}

Blanket Concept	ARIES-CS					SPPS
	Internal VV			External VV		External VV
	<u>Flibe/FS/Be</u>	<u>LiPb/SiC</u>	<u>LiPb/FS/He</u>	<u>LiPb/FS/He</u>	<u>Li/FS/He</u>	<u>Li/V</u>
<u>Thickness (cm):</u>						
WC Shield-I	33	25	52	52	67	—
FS Back Wall	—	—	11	9	8	—
Gap	1	1	—	1	1	—
WC Shield-II	25	36	—	26	10	—
Gap	2	2	2	—	—	—
LT Shield	—	—	—	—	—	—
VV	25	25	28	—	—	—
Total FW/S/VV	86	89	93	88	86	

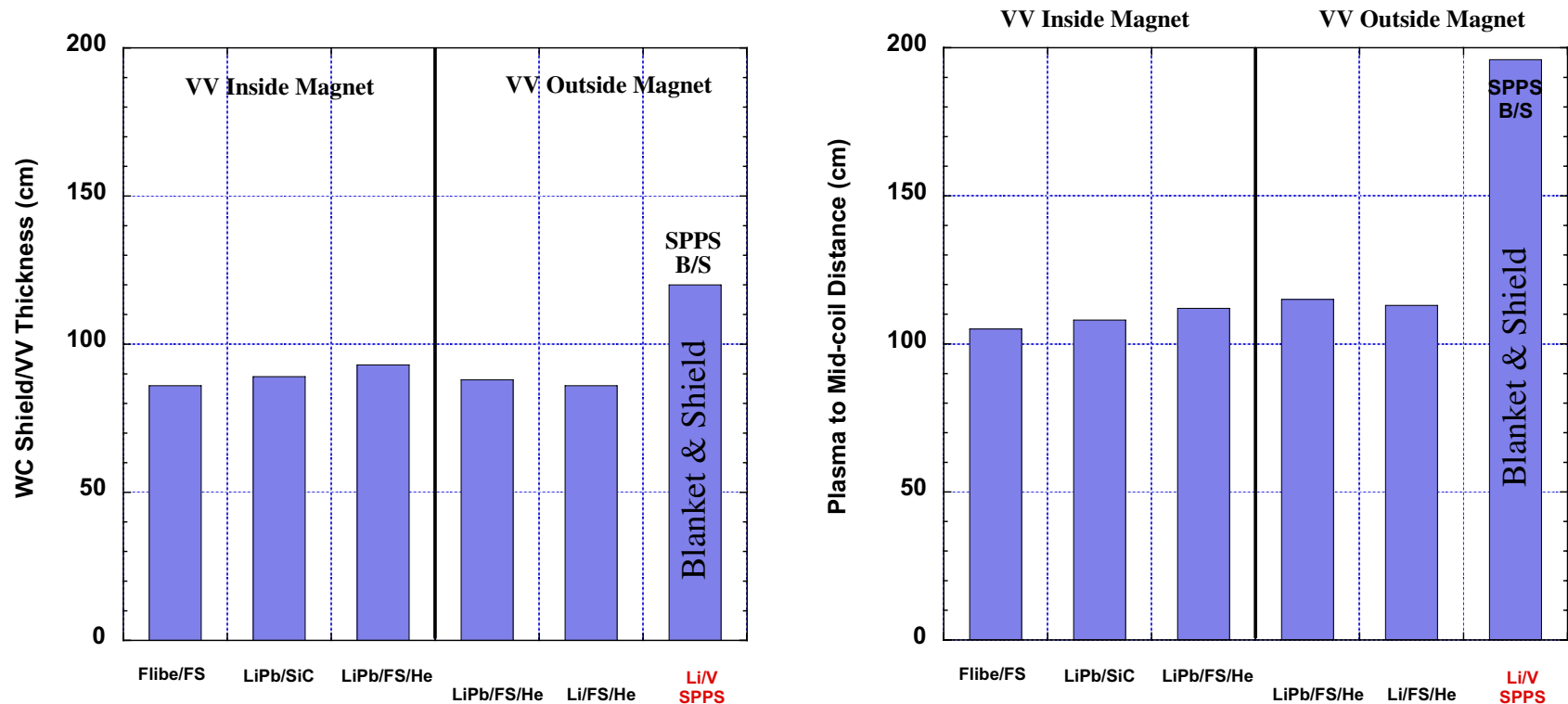
Dimensions of Δ_{\min} Shielding Components (no Blanket)

Blanket Concept	ARIES-CS					SPPS
	Internal VV			External VV		External VV
	<u>Flibe/FS/Be</u>	<u>LiPb/SiC</u>	<u>LiPb/FS/He</u>	<u>LiPb/FS/He</u>	<u>Li/FS/He</u>	<u>Li/V</u>
<u>Thickness (cm):</u>						
SOL	5	5	5	5	5	—
FW/S/VV	86	89	93	78	86	—
Gap + Th. Insl.	≥ 2	≥ 2	≥ 2	≥ 2	≥ 2	—
Cryostat*	2.2	2.2	2.2	10.2	10.2	—
1/2 Winding Pack	10	10	10	10	10	—
Δ_{\min}	105	108	112	115	113	

* Includes coil case and electric insulator.

Δ_{\min} varies slightly with blanket concept

Comparison Between Δ_{\min} (Shield-Only Zones, no Blanket)

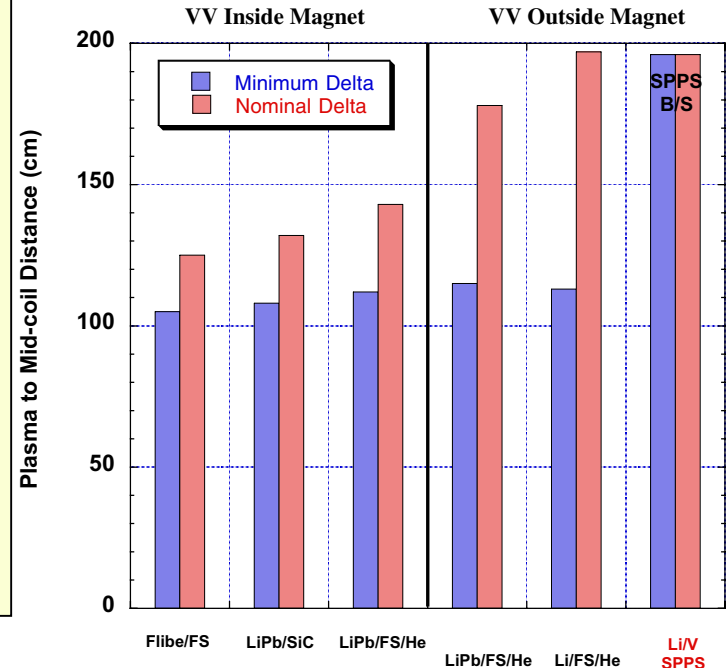


VV location has negligible impact on Δ_{\min}

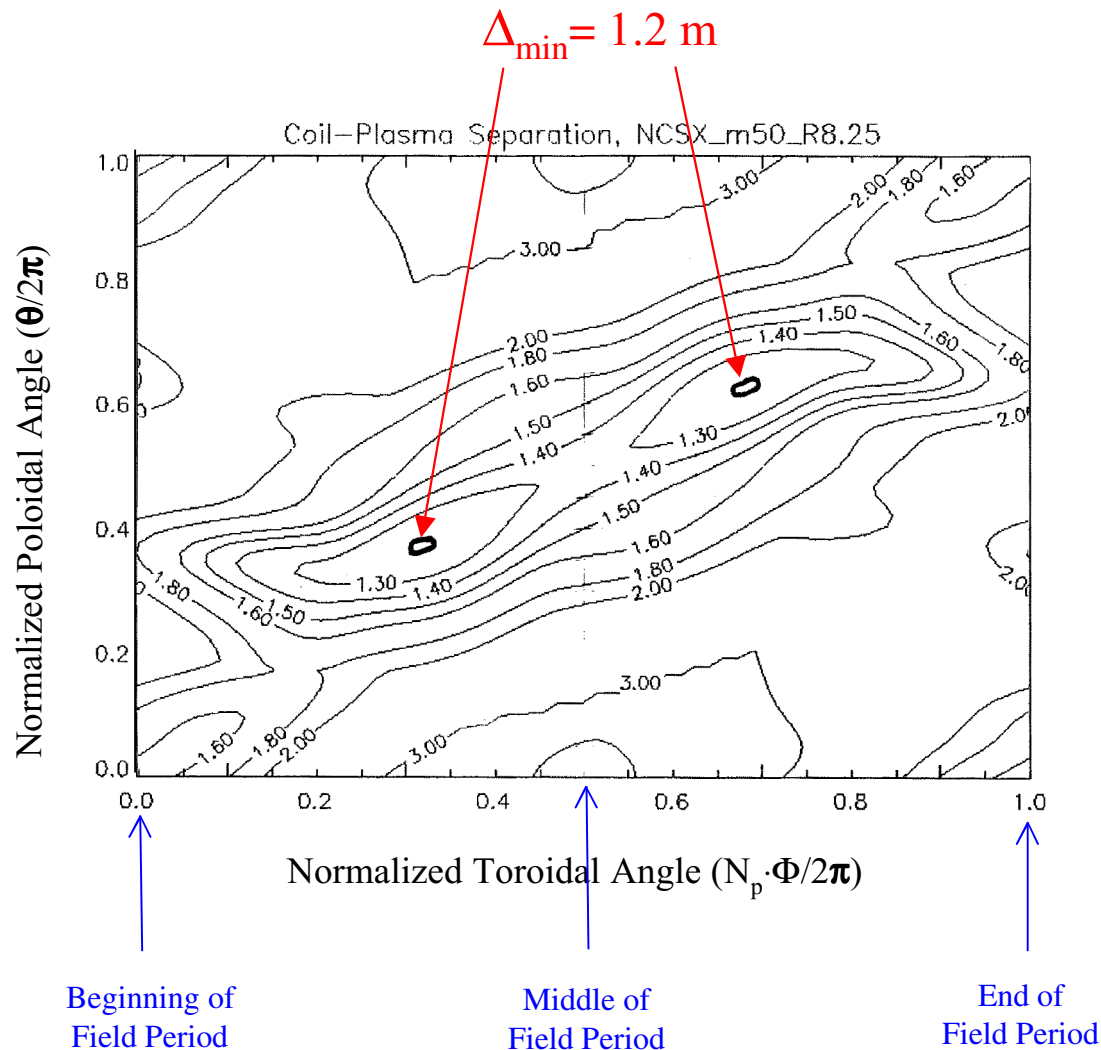
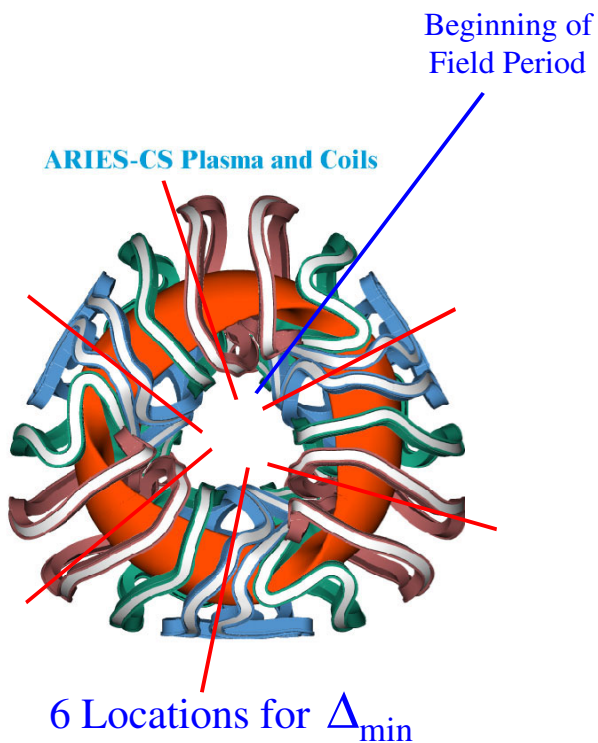
Comparison Between Radial Builds

Blanket Concept	ARIES-CS					SPPS
	Flibe/FS/Be	Internal VV LiPb/SiC	LiPb/FS/He	External VV LiPb/FS/He	External VV Li/FS/He	External VV Li/V
<u>Thickness (cm):</u>						
Nominal Δ	125	132	143	178	197	196
Δ_{\min} (shield-only)	105	108	112	115	113	—
Net Saving	20	24	31	63 ?	84 ?	—

- Shield-only zones could offer 20 - 84 cm (16 - 43%) reduction in radial build.
- Net saving exceeding 20 cm calls for shield-only zones with FW coverage > 8%, meaning overall TBR < 1.1.
- It is impossible to achieve TBR of 1.1 for LiPb and Li systems if shield-only zones cover 30-40% of FW area.
- Potential solutions:
 - For internal VV case, thicken LiPb blanket
 - For external VV case, install LiPb (or Li) blanket everywhere with uniform or variable thickness.



Δ_{\min} Occurs Twice per Field Period



Transition Region for Δ of 1.2-1.4 m

Covers ~8% of FW Area

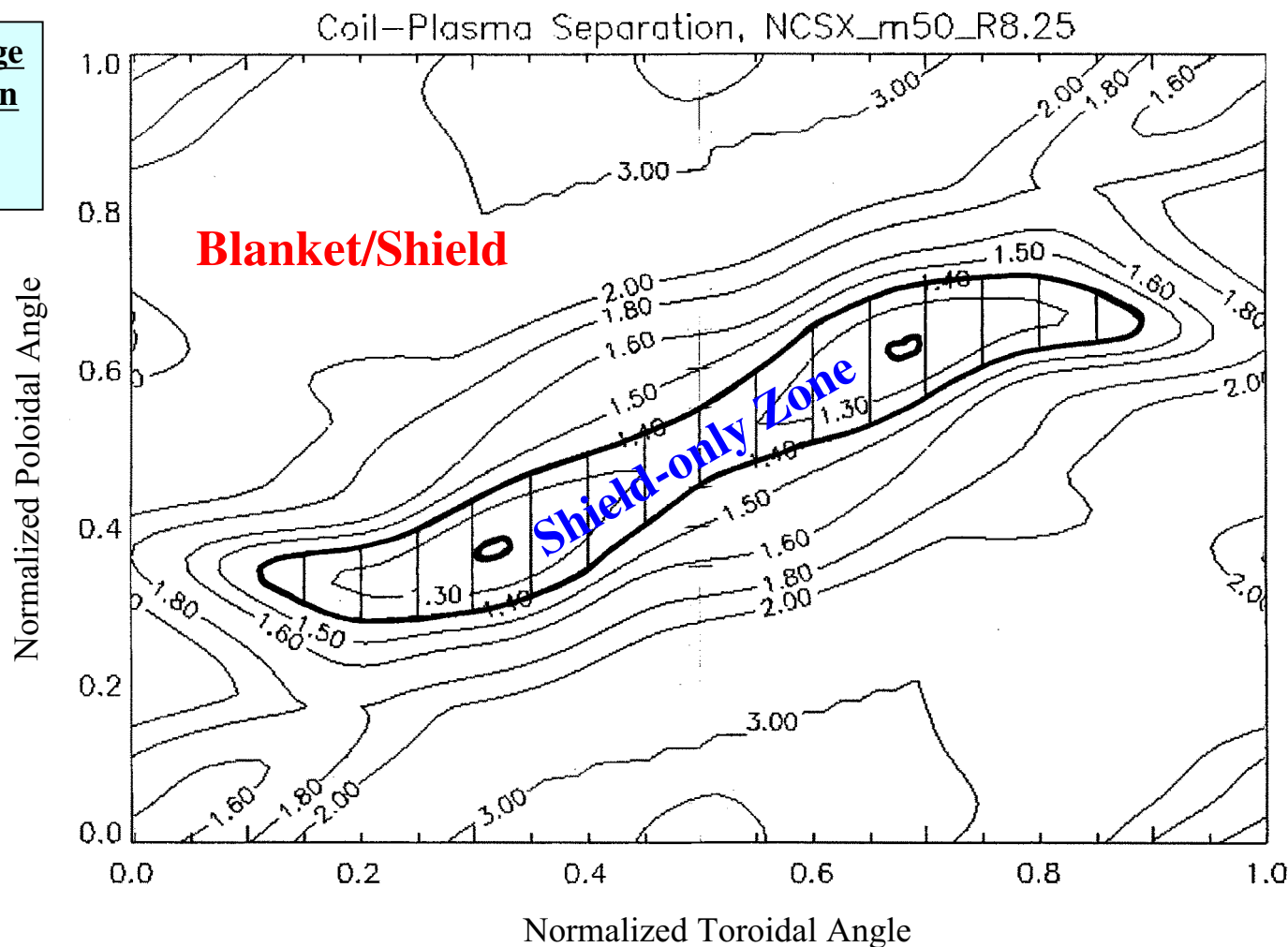
**Coverage
Fraction**

Shield-only zone

8%

Blanket/Shield

92%



Transition Region for Δ of 1.2-2.0 m

Covers ~40% of FW Area

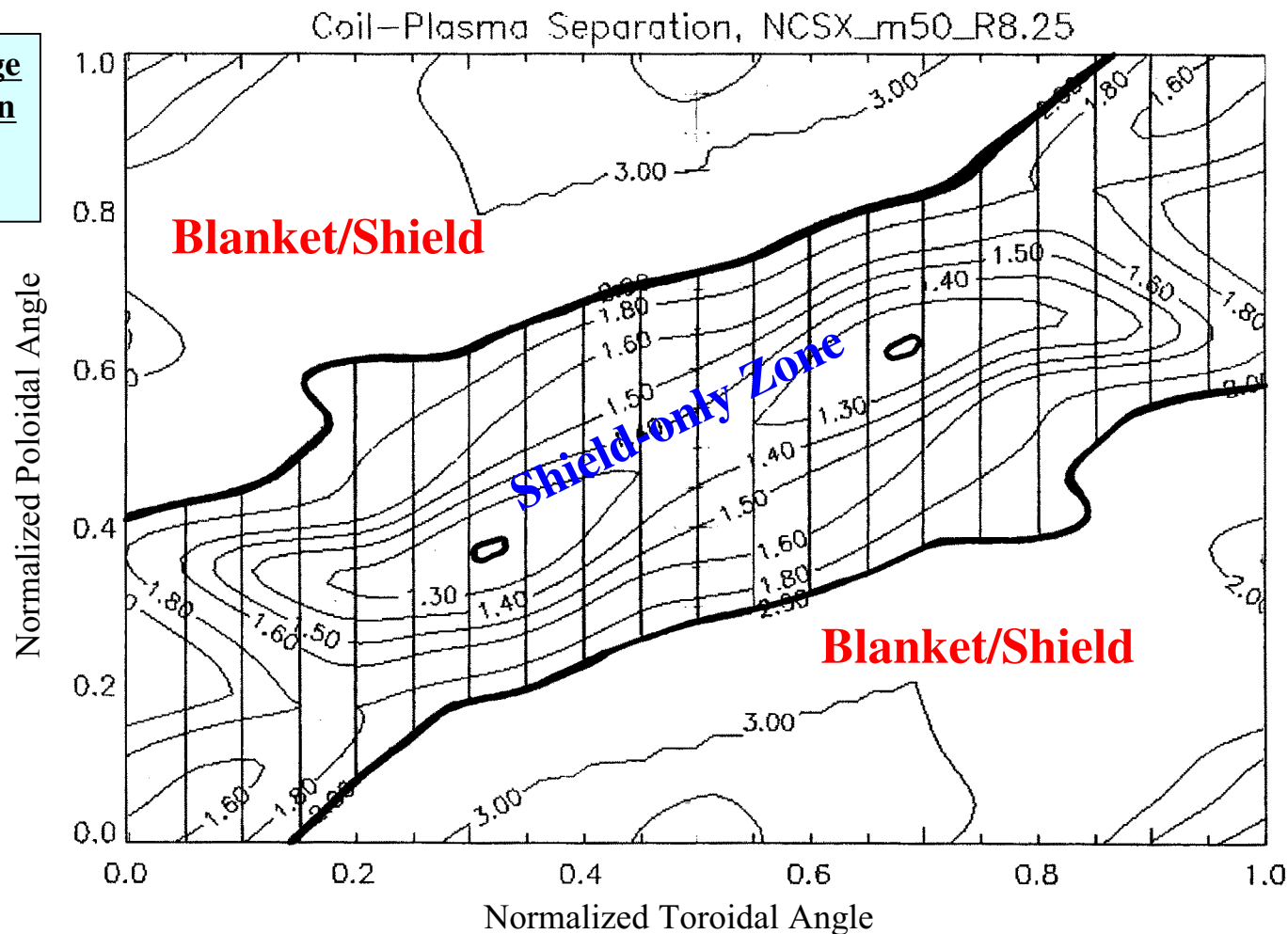
**Coverage
Fraction**

Shield-only zone

40%

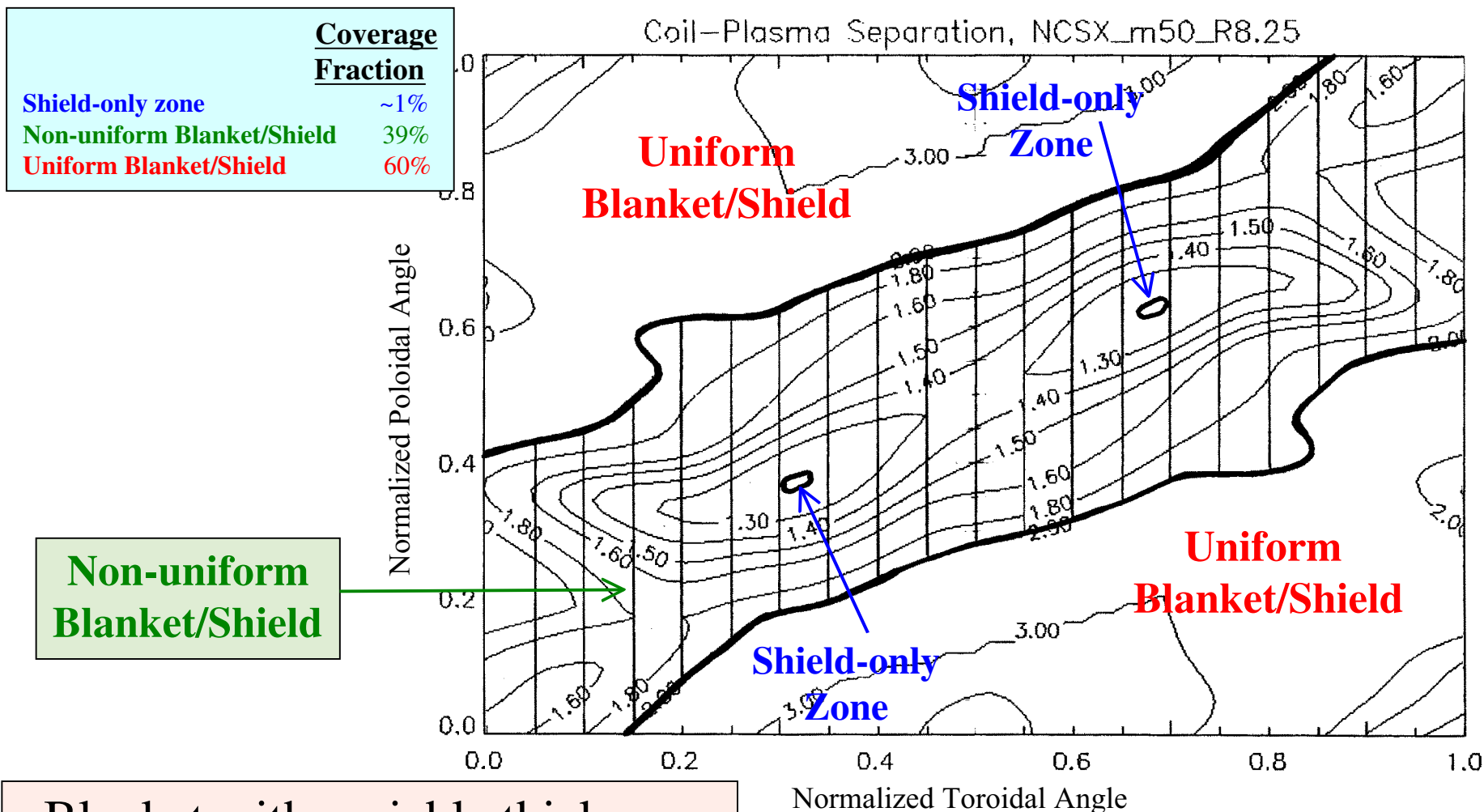
Blanket/Shield

60%



Blankets may not provide $TBR \geq 1.1$ if coverage fraction drops below 70%

Potential Solution for Sizable Transition Regions



WC-Shield-Only Zones for Δ_{\min} Introduce Engineering Problems

- Benefits:

- Compact radial build
- Smaller R and lower B_{\max}
- Lower COE

- Potential engineering problems:

- Integration of WC-shield with blanket system
- Accommodation of WC decay heat removal loop
- Heavy WC shield \Rightarrow many small modules, if weight-limited
- Any WC high level waste?

Key Parameters for ARIES-CS System Analysis

	<u>Flibe/FS/Be</u>	<u>LiPb/SiC</u>	<u>LiPb/FS</u>	<u>Li/FS</u>
TBR	1.1	1.1	1.1* ?	1.1* ?
Energy Multiplication (M_n)	1.2	1.1	1.15	1.13
Thermal Conversion Efficiency (η_{th})	45%	55-60%	~45%	~45%
FW Lifetime (FPY)	6.5	6	5	7
System Availability	~80%	~80%	~80%	~80%

* Based on 8% FW coverage fraction for shield-only zones.

Integrated system analysis could assess impact of
 M_n and η_{th} on COE for comparable Δ_{min}

Conclusions

- Radial standoff controls COE, a unique feature for stellarators.
- Employ expensive, highly efficient **local** shield at Δ_{\min} and combine **blanket with variable thickness** and cheap shield elsewhere.
- Blankets should breed extra tritium to compensate for losses due to non-breeding shield-only zones.
- To reduce Δ_{\min} , use:
 - WC-shield
 - Liquid breeder to cool HT shield
 - Water to cool LT shield (or internal VV)
 - No He coolant for shield.
- Shield-only Δ_{\min} varies slightly (2-7 cm) with blanket concept.
- External VV increases Δ_{\min} by few cm.
- Economic impact of shield-only zones needs to be assessed and innovative solutions to WC problems should be developed.

Conclusions (cont.)

- For water-cooled internal VV case, Flibe system offer most compact radial build, followed by LiPb.
- For external VV case, LiPb and Li blankets should be installed everywhere to meet breeding requirement.
- Water is superior shielding material that helps reduce nominal radial build and minimize radwaste volume
 - ⇒ **Exclude breeders incompatible with water.**
- **3-D nuclear analysis is needed to:**
 - Generate neutron wall loading profile using CAD-MCNP interface*
 - Assess 3-D impact of shield-only zones on breeding
 - Determine 3-D effect of penetrations, assembly gaps, and divertor system on TBR
 - Evaluate overall TBR for blankets with variable thickness
 - Confirm key nuclear parameters (M_n , lifetime, Δ_{\min} , etc)
 - Determine boundary between replaceable and permanent components.

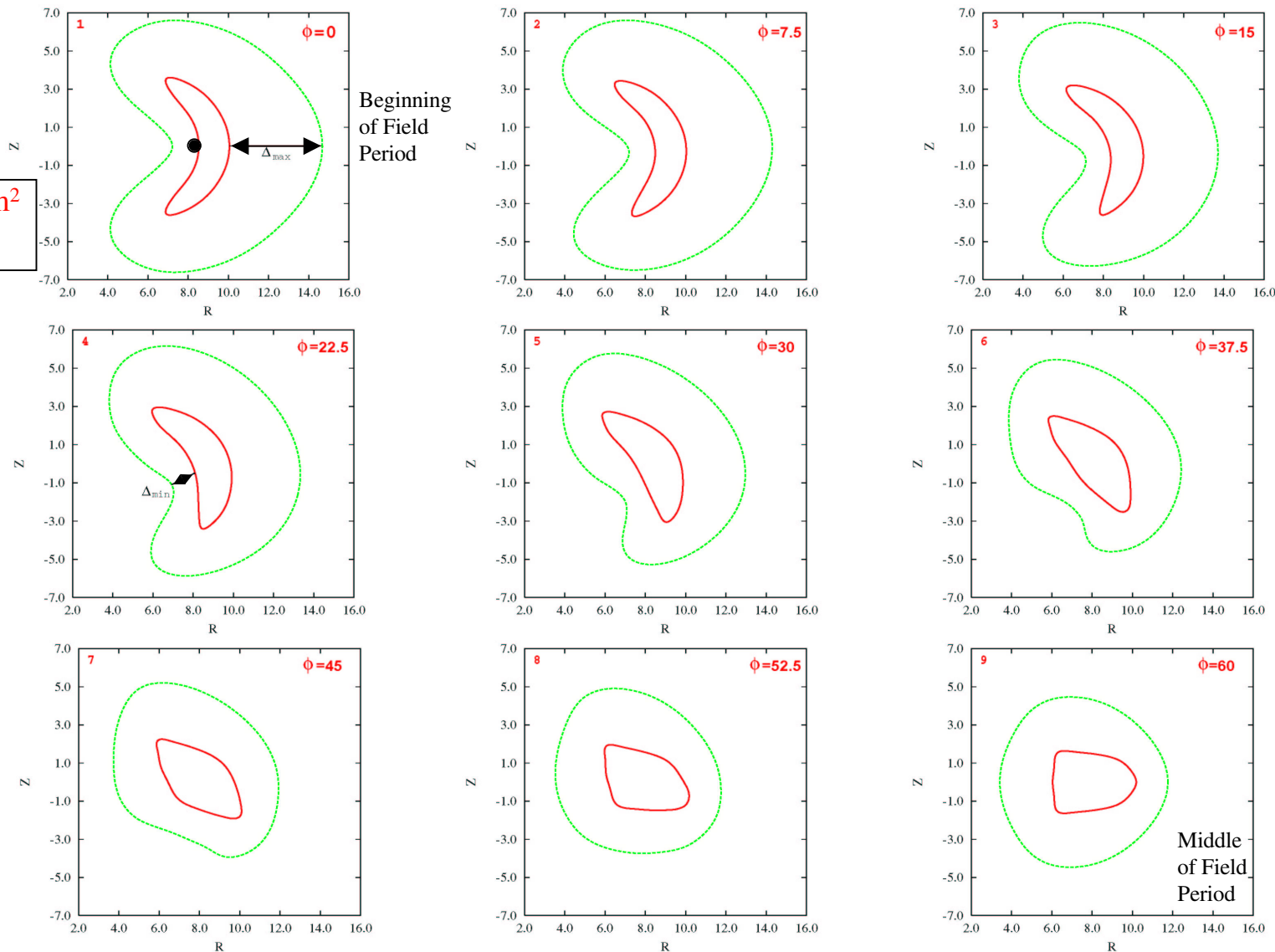
* Under development at UW.

9 Xns of Plasma Boundary (red) and WP Center (green) Covering Half Field Period (~9 m)

$\Delta_{\max} \sim 4.6$ m

Γ peaks @ ~ 3 MW/m²
at black dot

$\Delta_{\min} = 1.2$ m



Transition Region Between Δ_{\min} and $\Delta_{\min} + 0.2$ m Covers ~8% of FW Area

Beginning
of Field
Period

$$\Delta_{\min} = 1.2 \text{ m}$$

Blue areas:

~2 m Poloidal extent
~5.5 m Toroidal extent
~8% of FW area

